

AN ABSTRACT OF THE THESIS OF

R. Wayne Hardie for the Master of Science Degree in

Nuclear Engineering presented on *February 28, 1969*

Title: PERT-IV, A Two-Dimensional Perturbation Theory Code

for Fast Reactors *Redacted for Privacy*

Abstract approved:

Dr. Alan H. Robinson

A two-dimensional perturbation theory computer code, PERT-IV, has been developed that will calculate reactivity coefficients, the delayed neutron fraction, and the neutron generation time. The program uses the output flux and adjoint flux from either a diffusion theory or transport theory program. A discussion and derivation of the perturbation equation and expressions for the effective neutron delayed fraction and the neutron generation time are given. In addition, the reactivity coefficients as calculated by diffusion perturbation theory and  $S_N$  transport theory are compared.

The input flux and adjoint flux to PERT-IV can be taken directly from two-dimensional calculations or synthesized (by the code) from one-dimensional calculations. The code is compatible with the one-dimensional DTF-IV and two-dimensional 2DF transport theory codes from Los Alamos and the one-dimensional 1DX

and two-dimensional 2DB diffusion theory codes from Battelle-  
Northwest.

PERT-IV, A Two-Dimensional Perturbation  
Theory Code for Fast Reactors

by

Ralph Wayne Hardie

A THESIS

submitted to

Oregon State University

in partial fulfillment of

the requirements for the

degree of

Master of Science

June 1969

APPROVED:

*Redacted for Privacy*

---

Associate Professor of Nuclear Engineering

*Redacted for Privacy*

---

Head of Mechanical, Industrial, and Nuclear Engineering Department

*Redacted for Privacy*

---

Dean of Graduate School

Date thesis is presented

February 28, 1969

Typed by Judi Kasko for Ralph Wayne Hardie

## ACKNOWLEDGEMENTS

This report is based upon work performed by the author in the course of his employment with the Pacific Northwest Laboratory, operated by Battelle Memorial Institute for the Atomic Energy Commission under Contract AT(45-1)-1830. Use of this material as a basis for this thesis was approved by Oregon State University and the Pacific Northwest Laboratory. The author gratefully acknowledges the assistance of Dr. W. W. Little, Jr., of Pacific Northwest Laboratory and Dr. A. H. Robinson of Oregon State University. Finally, the author wishes to thank Mrs. Judi Kasko for typing the final copy.

## TABLE OF CONTENTS

I.	Introduction . . . . .	1
	Discussion of Perturbation Theory . . . . .	1
	PERT-IV, A Two-Dimensional Perturbation Code . . . . .	2
	Outline of Thesis . . . . .	4
II.	Derivation of Perturbation Equation. . . . .	5
III.	Derivation of Neutron Generation Time and Beta Effective Equations . . . . .	16
IV.	General Description of PERT-IV . . . . .	24
	Limits on Problem Complexity . . . . .	24
	Format of Input Fluxes and Adjoint Fluxes . . . . .	25
	Flux Normalization Options . . . . .	25
	Reactivity Coefficient Calculations . . . . .	26
	Neutron Generation Time and Beta Effective Calculations . .	26
	Activity Traverses . . . . .	26
	Other Options . . . . .	26
	Typical Results . . . . .	27
V.	Comparison Between Diffusion Perturbation Theory and Transport Theory . . . . .	29
VI.	Summary and Conclusions . . . . .	35
	Bibliography . . . . .	36
	Appendix I . . . . .	38
	Appendix II . . . . .	41
	Appendix III . . . . .	77

PERT-IV, A Two-Dimensional Perturbation  
Theory Code for Fast Reactors

I. INTRODUCTION

Perturbation Theory in General

Nuclear Engineers often desire to calculate the change in the effective multiplication constant,  $k_{\text{eff}}$ , caused by small changes in the macroscopic cross sections of a nuclear reactor. This is necessary, for example, in the operational calculations of a reactor such as the Fast Test Reactor<sup>(2)</sup> where numerous samples will be inserted into the core at various test positions. Another example of the usefulness of computing reactivity coefficients ( $\delta k_{\text{eff}}/k_{\text{eff}}$  per mass) is in the analysis of critical assemblies (e.g., ZPR-III, ZPR-VI<sup>(12)</sup>) where material worths are frequently measured for the purpose of checking cross section data and models.

One method of determining reactivity coefficients is to use a diffusion or transport theory computer code and calculate  $k_{\text{eff}}$  twice--with and without the sample. The disadvantage of this method is that a separate computer run is required for each reactivity coefficient at each position. Since a typical five energy group, two-dimensional transport theory problem can take from 15 to 30 minutes of UNIVAC 1108 computer time, the calculation of more than a couple material worths becomes very expensive. Moreover, since the change in  $k_{\text{eff}}$  is usually quite

small, the problem must be well converged for accurate worth calculations. Perturbation theory offers the advantage that once the fluxes and adjoint fluxes are known for the unperturbed case, many reactivity coefficients can be calculated very rapidly. A secondary benefit of perturbation theory is that it lends an insight into the causes of the change in  $k_{\text{eff}}$  due to a variation in the reactor composition because the perturbation equation gives the reactivity coefficient as the sum of four components: fission, absorption, leakage, and moderation.

It should be mentioned at this point that the use of perturbation theory is not a panacea for all material worth calculations. The main disadvantage is that perturbation theory assumes the variation in the reactor composition is "small". Frequently, the variation is either obviously too large for perturbation theory to work well--the insertion of a high worth control rod, for example--or it is unclear whether the variation is too large. Large perturbations in the flux caused by the variation result in large errors in the worth calculations. Therefore, if very accurate worths are desired for samples big enough to significantly perturb the flux, perturbation theory should not be used.

#### PERT-IV, A Two-Dimensional Perturbation Code

PERT-IV uses flux and adjoint flux output (in the form of punched cards or a binary tape) from diffusion or transport theory codes (e.g., 1DX<sup>(7)</sup>, 2DB<sup>(10)</sup>, DTF-IV<sup>(8)</sup>, 2DF<sup>(11)</sup>) and calculates



reactivity coefficient traverses for any specified row or column in the reactor using first order perturbation theory based on the multigroup diffusion model. If one-dimensional fluxes are used, both an axial and a radial case must be run for the flux and adjoint; the code then synthesizes the fluxes into a two-dimensional array. Each component of the perturbation equation, in addition to the total reactivity coefficient, is printed at each mesh point in the specified row or column.

Input to PERT-IV, in addition to the fluxes, consists of cross sections, axial and radial mesh structure, fission source data, and cross section mixture information. If the absolute number of neutrons per fission for each fissionable isotope, the delayed fission source, and neutron velocities are included, the neutron generation time,  $\Lambda$ , and the effective delayed neutron fraction,  $\beta_{\text{eff}}$ , are also calculated. This is a reasonable addition to a perturbation theory code because the expressions for  $\Lambda$  and  $\beta_{\text{eff}}$  are similar in many respects to the perturbation equation.

In a 65K memory (and 4 tapes), the code can handle up to 35 materials, 26 energy groups, and a 50 x 50 spatial mesh. The code will calculate up to 20 reactivity coefficient traverses in one run. The running time for a typical run (5 groups) on the UNIVAC 1108 is roughly 20 seconds.

Outline of Thesis

A derivation of the perturbation equation is given in Section II followed by a derivation of the expressions for the neutron generation time and beta effective in Section III. Section IV gives a general description of PERT-IV. Finally, a comparison is given in Section V between worths calculated by first order perturbation theory (PERT-IV), diffusion theory (2DB), and transport theory (DTF-IV and 2DF). Input instructions, a sample problem, and a FORTRAN-IV source deck listing are given in the Appendices.

## II. DERIVATION OF PERTURBATION EQUATION

Perturbation theory as applied to reactor physics was developed before the days of high speed computers by Wigner<sup>(13)</sup> to derive expressions for the change in the pile period due to cross-section changes in the two group equations for a bare, graphite pile. Another early contributor to the perturbation method was Brooks,<sup>(4)</sup> who derived an expression for the change in reactivity resulting from changes in the cross sections in the multigroup equations for a pile.

To derive the perturbation equation, we shall begin with the familiar multigroup time independent diffusion equation for an unperturbed system.

$$\begin{aligned} \bar{\nabla} \cdot D_i \bar{\nabla} \phi_i - \Sigma_a^i \phi_i - \sum_{j=i+1}^N \Sigma(i \rightarrow j) \phi_j + \frac{\chi_i}{k} \sum_{j=1}^N (v \Sigma_f)^j \phi_j \\ + \sum_{j=1}^{i-1} \Sigma(j \rightarrow i) \phi_j = 0 \quad , \end{aligned} \quad (2.1)$$

where:

$N$  = number of energy groups,

$\phi_i$  = flux in group  $i$  ( $n\text{-cm}^{-2}\text{-sec}^{-1}$ ),

$\chi_i$  = fission source born in group  $i$  ( $\sum_{i=1}^N \chi_i = 1.0$ ),

$\Sigma_a^i$  = macroscopic absorption cross section in group  $i$  ( $\text{cm}^{-1}$ ),

$D_i$  = diffusion coefficient for energy group  $i$  (cm),

$\Sigma(i \rightarrow j)$  = macroscopic transfer cross section from group  $i$   
to group  $j$  ( $\text{cm}^{-1}$ ),

$k$  = effective multiplication factor,

$(\nu\Sigma_f)^i$  = macroscopic fission source cross section in  
group  $i$  ( $\text{cm}^{-1}$ ).

Although the flux and cross sections are a function of position,  
for simplicity, the spatial variables are not explicitly shown.

A similar expression can also be written for a perturbed  
system.

$$\begin{aligned} \bar{\nabla} \cdot D_i' \bar{\nabla} \phi_i' - \Sigma_a^{i'} \phi_i' - \sum_{j=i+1}^N \Sigma'(i \rightarrow j) \phi_j' + \frac{\chi_i}{k'} \sum_{j=1}^N (\nu\Sigma_f)^j \phi_j' \\ + \sum_{j=1}^{i-1} \Sigma'(j \rightarrow i) \phi_j' = 0 \quad , \end{aligned} \quad (2.2)$$

where:  $\Sigma^{i'} = \Sigma^i + \delta\Sigma^i$  for  $\Sigma_a^{i'}$ ,  $\Sigma_f^{i'}$  and  $\Sigma'(i \rightarrow j)$  ,

$$D_i' = D_i + \delta D_i \quad ,$$

$$\phi_i' = \phi_i + \delta\phi_i \quad ,$$

$$k' = k + \delta k \quad .$$

First, we multiply Equations (2.1) and (2.2) by an arbitrary  
weighting function  $\psi_i$  (also a function of spatial position), sum over  
all groups, integrate over the entire volume of the reactor, and sub-  
tract the first integral equation from the second. This yields

$$\begin{aligned}
& \int_V \sum_{i=1}^N \psi_i (\bar{\nabla} \cdot D_i \bar{\nabla} \phi_i' - \bar{\nabla} \cdot D_i \bar{\nabla} \phi_i) dV - \int_V \sum_{i=1}^N \psi_i (\Sigma_a^{i'} \phi_i' - \Sigma_a^i \phi_i) dV \\
& - \int_V \sum_{i=1}^N \psi_i \left( \sum_{j=i+1}^N \Sigma'(i \rightarrow j) \phi_i' - \sum_{j=i+1}^N \Sigma(i \rightarrow j) \phi_i \right) dV \\
& + \int_V \left[ \sum_{i=1}^N \frac{\psi_i \chi_i}{k'} \sum_{j=1}^N (v \Sigma_f)^j \phi_j' - \sum_{i=1}^N \frac{\psi_i \chi_i}{k} \sum_{j=1}^N (v \Sigma_f)^j \phi_j \right] dV \\
& + \int_V \sum_{i=1}^N \psi_i \left[ \sum_{j=1}^{i-1} \Sigma'(j \rightarrow i) \phi_j' - \sum_{j=1}^{i-1} \Sigma(j \rightarrow i) \phi_j \right] dV = 0 \quad , \quad (2.3)
\end{aligned}$$

or, 
$$I_1 + I_2 + I_3 + I_4 + I_5 = 0 \quad , \quad (2.4)$$

where  $I_1, I_2, I_3, I_4,$  and  $I_5$  are the individual integrals.

The next step is to expand the primed quantities in Equation (2.3) and consider each integral separately. First order perturbation theory consists of ignoring all terms higher than first order. That is,

$$\Sigma' \phi' = (\Sigma + \delta \Sigma)(\phi + \delta \phi) \approx \Sigma \phi + \phi \delta \Sigma + \Sigma \delta \phi \quad .$$

Expanding the diffusion coefficient and fluxes to the first order and then cancelling the second term in the first integral leaves

$$I_1 = \int_V \sum_{i=1}^N \psi_i [\bar{\nabla} \cdot \delta D_i \bar{\nabla} \phi_i + \bar{\nabla} \cdot D_i \bar{\nabla} \delta \phi_i] dV \quad . \quad (2.5)$$

Equation (2.5) can also be written as

$$I_1 = \delta \int_V \sum_{i=1}^N \psi_i [\bar{\nabla} \cdot D_i \bar{\nabla} \phi_i] dV \quad . \quad (2.6)$$

To evaluate the above equation, we will make use of Green's theorem in first and second form.

$$\int_V D \bar{\nabla} \phi \cdot \bar{\nabla} \psi dV = \int_S \phi D \bar{\nabla} \psi \cdot d\bar{S} - \int_V \phi \bar{\nabla} \cdot D \bar{\nabla} \psi dV \quad , \quad (2.7)$$

and

$$\int_V D \bar{\nabla} \phi \cdot \bar{\nabla} \psi dV = \int_S \psi D \bar{\nabla} \phi \cdot d\bar{S} - \int_V \psi \bar{\nabla} \cdot D \bar{\nabla} \phi dV \quad . \quad (2.8)$$

Applying Green's theorem in second form to Equation (2.6),

$$I_1 = \delta \int_S \sum_{i=1}^N \psi_i D_i \bar{\nabla} \phi_i \cdot d\bar{S} - \delta \int_V \sum_{i=1}^N D_i \bar{\nabla} \phi_i \cdot \bar{\nabla} \psi_i dV \quad . \quad (2.9)$$

Now, since diffusion theory assumes that the current,  $D_i \bar{\nabla} \phi_i$ , is a continuous function across an interface of two dissimilar media,  $D_i \bar{\nabla} \phi_i$  is constant at the surface of the volume where the perturbation occurs. Therefore, the first term in Equation (2.9) is zero. This assumes, of course, that  $\psi_i$  is also continuous across the interface. To circumvent the case when the perturbation is over the whole reactor and the interface is not between two media, we require that  $\psi_i$  vanish at the reactor boundary.

Taking the variation of the second term in Equation (2.9)

gives

$$I_1 = - \int_V \sum_{i=1}^N \delta D_i \bar{\nabla} \phi_i \cdot \bar{\nabla} \psi_i dV - \int_V \sum_{i=1}^N D_i \bar{\nabla} \delta \phi_i \cdot \bar{\nabla} \psi_i dV \quad . \quad (2.10)$$

Using Green's theorem in first form in the second term of Equation (2.10),

$$\begin{aligned} I_1 = & - \int_V \sum_{i=1}^N \delta D_i \bar{\nabla} \phi_i \cdot \bar{\nabla} \psi_i dV - \int_S \sum_{i=1}^N \delta \phi_i D_i \bar{\nabla} \psi_i \cdot d\bar{S} \\ & + \int_V \sum_{i=1}^N \delta \phi_i \bar{\nabla} \cdot D_i \bar{\nabla} \psi_i dV \quad . \end{aligned} \quad (2.11)$$

The first term in Equation (2.9) was found to be equal to zero because the current is continuous across an interface. Similarly, the flux is a continuous function causing the surface integral in Equation (2.11) to also vanish.

Therefore, we conclude that

$$I_1 = - \int_V \sum_{i=1}^N \delta D_i \bar{\nabla} \phi_i \cdot \bar{\nabla} \psi_i dV + \int_V \sum_{i=1}^N \delta \phi_i \bar{\nabla} \cdot D_i \bar{\nabla} \psi_i dV \quad . \quad (2.12)$$

The second integral in Equation (2.3) is simply

$$I_2 = - \int_V \sum_{i=1}^N (\psi_i \phi_i \delta \Sigma_a^i) dV - \int_V \sum_{i=1}^N (\psi_i \Sigma_a^i \delta \phi_i) dV \quad . \quad (2.13)$$

To evaluate the fourth integral, we note that for small  $\delta k$ ,

$$\frac{1}{k'} = \frac{1}{k + \delta k} \approx \frac{1}{k} \left(1 - \frac{\delta k}{k}\right) . \quad (2.14)$$

Therefore,

$$\begin{aligned} I_4 &= \int_V \frac{1}{k} \left[ \sum_{i=1}^N \chi_i \psi_i \sum_{j=1}^N \phi_j \delta(v \Sigma_r)^j \right] dV \\ &\quad - \frac{\delta k}{k} \int_V \frac{1}{k} \left[ \sum_{i=1}^N \chi_i \psi_i \sum_{j=1}^N (v \Sigma_r)^j \phi_j \right] dV \\ &\quad + \int_V \frac{1}{k} \left[ \sum_{i=1}^N \chi_i \psi_i \sum_{j=1}^N (v \Sigma_r)^j \delta \phi_j \right] dV . \end{aligned} \quad (2.15)$$

Since the third and fifth integrals are somewhat related, we shall work on them together.

$$\begin{aligned} I_3 + I_5 &= - \int_V \left[ \sum_{i=1}^N \psi_i \sum_{j=i+1}^N \phi_i \delta \Sigma(i \rightarrow j) \right] dV \\ &\quad - \int_V \left[ \sum_{i=1}^N \psi_i \sum_{j=i+1}^N \Sigma(i \rightarrow j) \delta \phi_i \right] dV \\ &\quad + \int_V \left[ \sum_{i=1}^N \psi_i \sum_{j=1}^{i-1} \phi_j \delta \Sigma(j \rightarrow i) \right] dV \\ &\quad + \int_V \left[ \sum_{i=1}^N \psi_i \sum_{j=1}^{i-1} \Sigma(j \rightarrow i) \delta \phi_j \right] dV . \end{aligned} \quad (2.16)$$



To simplify the above expression, we consider the following double summation,

$$DS = \sum_{i=1}^N \psi_i \sum_{j=1}^{i-1} \phi_j \Sigma(j \rightarrow i) \quad . \quad (2.17)$$

If we put  $\psi_i$  inside the second summation, reverse the order of the summation, and note that  $i > j$ ,

$$DS = \sum_{j=1}^N \sum_{i=j+1}^N \psi_i \Sigma(j \rightarrow i) \phi_j \quad . \quad (2.18)$$

Finally, interchanging the indices and pulling  $\phi_i$  outside the second summation results in the following identity:

$$\sum_{i=1}^N \psi_i \sum_{j=1}^{i-1} \phi_j \Sigma(j \rightarrow i) = \sum_{i=1}^N \phi_i \sum_{j=i+1}^N \psi_j \Sigma(i \rightarrow j) \quad . \quad (2.19)$$

The above equation enables us to write Equation (2.16) as

$$\begin{aligned} I_3 + I_5 &= \int_V \sum_{i=1}^N \phi_i \sum_{j=i+1}^N \delta \Sigma(i \rightarrow j) (\psi_j - \psi_i) dV \\ &+ \int_V \sum_{i=1}^N \delta \phi_i \left( \sum_{j=i+1}^N \Sigma(i \rightarrow j) \psi_j - \sum_{j=i+1}^N \Sigma(i \rightarrow j) \psi_i \right) dV \quad . \quad (2.20) \end{aligned}$$

Substituting the values for  $I_1$ ,  $I_2$ ,  $I_3$ ,  $I_4$ , and  $I_5$  in Equation (2.4) and solving for  $\delta k/k$ , we get

$$\begin{aligned}
\frac{\delta k}{k} = \int_V \left\{ \sum_{i=1}^N \left[ \frac{1}{k} \delta(\nu \Sigma_f)^i \phi_i \sum_{j=1}^N \chi_j \psi_j - \delta \Sigma_a^i \phi_i \psi_i \right. \right. \\
- \delta D_i (\overline{\nabla \phi_i} \cdot \overline{\nabla \psi_i}) + \sum_{j=i+1}^N \delta \Sigma(i \rightarrow j) \phi_i (\psi_j - \psi_i) \\
+ \delta \phi_i \left( \overline{\nabla} \cdot D_i \overline{\nabla \psi_i} - \Sigma_a^i \psi_i - \sum_{j=i+1}^N \Sigma(i \rightarrow j) \psi_i + \sum_{j=i+1}^N \Sigma(i \rightarrow j) \psi_j \right. \\
\left. \left. + \frac{(\nu \Sigma_f)^i}{k} \sum_{j=1}^N \chi_j \psi_j \right) \right\} dV / \frac{1}{k} \int_V \sum_{i=1}^N \chi_i \psi_i \sum_{j=1}^N (\nu \Sigma_f)^j \phi_j dV.
\end{aligned} \tag{2.21}$$

Now, we note that if the coefficient of  $\delta \phi_i$  is equal to zero, then the variation in the flux doesn't contribute to the perturbation equation. Therefore, we define the weighting function as

$$\begin{aligned}
\overline{\nabla} \cdot D_i \overline{\nabla \psi_i} - \Sigma_a^i \psi_i - \sum_{j=i+1}^N \Sigma(i \rightarrow j) \psi_i + \sum_{j=i+1}^N \Sigma(i \rightarrow j) \psi_j \\
+ \frac{(\nu \Sigma_f)^i}{k} \sum_{j=1}^N \chi_j \psi_j = 0.
\end{aligned} \tag{2.22}$$

Observe that this is simply the adjoint equation. The relationship between the diffusion equation, Equation (2.1), and the adjoint equation is easily understood since if one writes the diffusion equation in matrix form,

$$\bar{M}\bar{\phi} = 0 \quad ,$$

the adjoint equation can be written as

$$\bar{M}^T\bar{\psi} = 0 \quad ,$$

where  $\bar{M}^T$  is the transpose of  $\bar{M}$ . It should be mentioned that, unlike the flux, the adjoint flux is dimensionless.

It is easy to show the physical significance of the adjoint flux. To do this, let  $\xi_i$  be the worth of a neutron in energy group  $i$ , where we have again not explicitly shown the spatial dependence.

The neutron worth, or total number of neutrons produced by inserting one neutron in energy group  $i$  is

$$\xi_i = \frac{(\nu\Sigma_f)^i\phi_i \sum_{j=1}^N \chi_j \xi_j + \sum_{j=i+1}^N \Sigma(i\rightarrow j)\phi_i \xi_j}{-\bar{\nabla}\cdot D_i \bar{\nabla}\phi_i + \Sigma_a^i\phi_i + \sum_{j=i+1}^N \Sigma(i\rightarrow j)\phi_i} \quad , \quad (2.23)$$

where the first term in the numerator is the number of neutrons emitted from fission multiplied by the worth of an averaged fission

neutron,  $\sum_{j=1}^N \chi_j \xi_j$ , and the second term is the scattering into group  $j$

multiplied by the worth in  $j$ . The denominator is just the loss rate.

We now clear Equation (2.23) of fractions, integrate over the volume of the reactor, and use Green's theorem on the leakage term. Since the flux and adjoint flux vanish on the boundary, we can cancel  $\phi_i$  from both sides and write

$$\begin{aligned} \bar{\nabla} \cdot D_i \bar{\nabla} \xi_i - \Sigma_a^i \xi_i - \sum_{j=i+1}^N \Sigma(i \rightarrow j) \xi_i + \sum_{j=i+1}^N \Sigma(i \rightarrow j) \xi_j \\ + \frac{(\nu \Sigma_f)^i}{k} \sum_{j=1}^N \chi_j \xi_j = 0 \quad . \end{aligned} \quad (2.24)$$

Comparing Equation (2.24) with Equation (2.22),  $\xi_i = \psi_i$  and the weighting function is simply the neutron worth or adjoint flux.

With this interpretation of the adjoint flux, it is now possible to assign a physical meaning to each of the four components of Equation (2.21), the perturbation equation. Changes in the neutron fission source rate are weighted by the worth of an average fission neutron while changes in the absorption rate are weighted by the neutron worth at the point of capture. The third term is the worth due to a change in the leakage rate.

Note that since  $D = \frac{1}{3\Sigma_{tr}}$ ,

$$\delta D_i = - \frac{1}{3(\Sigma_{tr}^i)^2} \delta(\Sigma_{tr}^i) \quad ,$$

where  $\Sigma_{tr}^i$  = macroscopic transport cross section in group  $i$  ( $\text{cm}^{-1}$ ). Finally, as seen in the fourth term, variations in  $\Sigma(i \rightarrow j)$  are weighted by the worth in group  $i$  minus the worth in group  $j$ .

III. DERIVATION OF NEUTRON GENERATION TIME  
AND BETA EFFECTIVE EQUATIONS

The time dependent diffusion equation for energy group  $i$  with NP delayed precursor groups can be written as

$$\begin{aligned} \frac{1}{v_i} \frac{\partial \phi_i}{\partial t} = & \bar{\nabla} \cdot D_i \bar{\nabla} \phi_i - \Sigma_a^i \phi_i - \Sigma_r^i \phi_i + \chi_i \sum_{j=1}^N (v \Sigma_f)^j \phi_j \\ & - \chi_i \sum_{j=1}^N \phi_j \sum_{m=1}^{ND} \beta_m (v \Sigma_f)_m^j + \chi_i \sum_{K=1}^{NP} \lambda_K C_K \\ & + \sum_{j=1}^{i-1} \Sigma(j \rightarrow i) \phi_j \quad , \end{aligned} \quad (3.1)$$

where:

$\phi_i$  = flux in group  $i$  ( $n\text{-cm}^{-2}\text{-sec}^{-1}$ ),

$v_i$  = neutron velocity in energy group  $i$  (cm/sec),

$\Sigma_r^i$  = macroscopic scattering removal cross section in group  $i$  ( $\text{cm}^{-1}$ ),

$\beta_m$  = number of delayed neutrons/fission neutrons for fissionable isotope  $m$ ,

ND = number of fissionable isotopes,

NP = number of precursor groups,

$(\nu\Sigma_f)_m^i$  = macroscopic fission source cross section in group  
i for fissionable isotope m ( $\text{cm}^{-1}$ ),

$\chi_i^D$  = delayed fission source born in group i

$$\sum_{i=1}^N \chi_i^D = 1.0),$$

$\lambda_K$  = decay constant for delayed precursor group K  
( $\text{sec}^{-1}$ ),

and  $C_K$  = density for delayed precursor group K ( $\text{cm}^{-3}$ ).

All other symbols in the above equation were defined after Equation (2.1) in the previous section. It is obvious from Equation (3.1) that the flux and precursor density are functions of both spatial position and time. Note the assumption is made that  $\chi_i^D$  is independent of the delayed precursor group.

The differential equation describing the time behavior of the Kth precursor group is

$$\frac{\partial C_K}{\partial t} = \sum_{j=1}^N \phi_j \sum_{m=1}^{ND} \beta_m^K (\nu\Sigma_f)_m^j - \lambda_K C_K, \quad (3.2)$$

where:

$\beta_m^K$  = delayed neutron fraction for precursor group K

for fissionable isotope m ( $\beta_m = \sum_{K=1}^{NP} \beta_m^K$ ).

From Section II, we saw that the time independent adjoint equation for energy group  $i$  is

$$0 = \bar{\nabla} \cdot D_i \bar{\nabla} \psi_i - \Sigma_a^i \psi_i - \Sigma_r^i \psi_i + \sum_{j=i+1}^N \Sigma(i \rightarrow j) \psi_j + \frac{(\nu \Sigma_f)^i}{k} \sum_{j=1}^N \chi_j \psi_j \quad (3.3)$$

Equation (3.1) is now multiplied by the adjoint flux and Equation (3.3) by the flux, both equations summed over all groups and integrated over the volume of the reactor. Subtracting the second resulting integral equation from the first,

$$\begin{aligned} & \int_V \sum_{i=1}^N \frac{\psi_i}{v_i} \frac{\partial \phi_i}{\partial t} dV = \int_V \sum_{i=1}^N \psi_i \bar{\nabla} \cdot D_i \bar{\nabla} \phi_i dV - \int_V \sum_{i=1}^N \psi_i \Sigma_a^i \phi_i dV \\ & - \int_V \sum_{i=1}^N \psi_i \Sigma_r^i \phi_i dV + \int_V \sum_{i=1}^N \chi_i \psi_i \sum_{j=1}^N (\nu \Sigma_f)^j \phi_j dV \\ & - \int_V \sum_{i=1}^N \chi_i \psi_i \sum_{j=1}^N \phi_j \sum_{m=1}^{ND} \beta_m (\nu \Sigma_f)_m^j dV + \int_V \sum_{i=1}^N \chi_i^D \psi_i \sum_{K=1}^{NP} \lambda_K C_K dV \\ & + \int_V \sum_{i=1}^N \psi_i \sum_{j=1}^{i-1} \Sigma(j \rightarrow i) \phi_j dV - \int_V \sum_{i=1}^N \phi_i \bar{\nabla} \cdot D_i \bar{\nabla} \psi_i dV \\ & + \int_V \sum_{i=1}^N \phi_i \Sigma_a^i \psi_i dV + \int_V \sum_{i=1}^N \phi_i \Sigma_r^i \psi_i dV - \int_V \sum_{i=1}^N \phi_i \sum_{j=i+1}^N \Sigma(i \rightarrow j) \psi_j dV \\ & - \int_V \sum_{i=1}^N \frac{(\nu \Sigma_f)^i \phi_i}{k} \sum_{j=1}^N \chi_j \psi_j dV \quad (3.4) \end{aligned}$$



To simplify Equation (3.4), we shall utilize the following relationships.

Applying Green's theorem,

$$\begin{aligned} \int_V \sum_{i=1}^N \phi_i \bar{\nabla} \cdot D_i \bar{\nabla} \psi_i dV &= \int_V \sum_{i=1}^N \psi_i \bar{\nabla} \cdot D_i \bar{\nabla} \phi_i dV \\ &+ \int_V \sum_{i=1}^N D_i (\phi_i \bar{\nabla} \psi_i - \psi_i \bar{\nabla} \phi_i) \cdot d\bar{S} \quad . \end{aligned} \quad (3.5)$$

Since the flux and adjoint flux both vanish at the reactor boundary, the second integral on the right side of the above equation is zero and we can write

$$\int_V \sum_{i=1}^N \phi_i \bar{\nabla} \cdot D_i \bar{\nabla} \psi_i dV = \int_V \sum_{i=1}^N \psi_i \bar{\nabla} \cdot D_i \bar{\nabla} \phi_i dV \quad . \quad (3.6)$$

Defining  $\rho = \text{reactivity} = \frac{k-1}{k}$  ,

$$\begin{aligned} \int_V \sum_{i=1}^N \chi_i \psi_i \sum_{j=1}^N (\nu \Sigma_f)^j \phi_j dV - \int_V \sum_{i=1}^N \frac{(\nu \Sigma_f)^i \phi_i}{k} \sum_{j=1}^N \chi_j \psi_j dV \\ = \rho \int_V \sum_{i=1}^N \chi_i \psi_i \sum_{j=1}^N (\nu \Sigma_f)^j \phi_j dV \quad . \end{aligned} \quad (3.7)$$

Substituting Equations (2.19), (3.6), and (3.7) into Equation (3.4) gives, after cancellation,

$$\begin{aligned}
 \int_V \sum_{i=1}^N \frac{\psi_i}{v_i} \frac{\partial \phi_i}{\partial t} dV &= \rho \int_V \sum_{i=1}^N \chi_i \psi_i \sum_{j=1}^N (\nu \Sigma_f)^j \phi_j dV \\
 &\quad - \int_V \sum_{i=1}^N \chi_i \psi_i \sum_{j=1}^N \phi_j \sum_{m=1}^{ND} \beta_m (\nu \Sigma_f)_m^j dV \\
 &\quad + \int_V \sum_{i=1}^N \chi_i^D \psi_i \sum_{K=1}^{NP} \lambda_K C_K dV \quad . \quad (3.8)
 \end{aligned}$$

We now multiply Equation (3.2) by  $\sum_{i=1}^N \chi_i^D \psi_i$  and integrate over the volume of the reactor.

$$\begin{aligned}
 \int_V \sum_{i=1}^N \chi_i^D \psi_i \frac{\partial C_K}{\partial t} dV &= \int_V \sum_{i=1}^N \chi_i^D \psi_i \sum_{j=1}^N \phi_j \sum_{m=1}^{ND} \beta_m^K (\nu \Sigma_f)_m^j dV \\
 &\quad - \int_V \sum_{i=1}^N \chi_i^D \psi_i \lambda_K C_K dV \quad . \quad (3.9)
 \end{aligned}$$

What is now desired is to reduce Equations (3.8) and (3.9) to the conventional form of the reactor kinetics equations,

$$\frac{dT(t)}{dt} = \frac{\rho - \beta}{\Lambda} T(t) + \sum_{K=1}^{NP} \lambda_K C_K \quad , \quad (3.10)$$

and

$$\frac{dC_K(t)}{dt} = \frac{\beta}{\Lambda} T(t) - \lambda_K C_K \quad . \quad (3.11)$$

As noted earlier, the flux in Equation (3.8) is a function of both space and time. For a homogeneous reactor, the spatial and time variables are separable and one can speak of the time behavior for the reactor as a whole. For a zoned reactor, however, the variables are not separable and it is necessary to express the flux as the product of two functions.

$$\phi_i(r,t) = \phi_i(r,t)T(t) \quad . \quad (3.12)$$

At first glance, it doesn't appear that this helps much. However, it is assumed that the nonseparable portion of the flux,  $\phi_i(r,t)$  becomes independent of time after a sufficiently large time following the perturbation. That is,

$$\lim_{t \rightarrow \infty} \phi_i(r,t) = \phi_i(r) \equiv \phi_i \quad . \quad (3.13)$$

Therefore,  $T(t)$  represents the asymptotic time behavior of the flux. A discussion of the transient neutron flux for a zoned reactor is given by Crawford, (5, p. 41-56) where, using diffusion theory, he expands the flux as a sum of model fluxes and writes the kinetics equations for each mode.

As a consequence of Equation (3.13), we write

$$\begin{aligned} \int_V \sum_{i=1}^N \frac{\psi_i}{v_i} \frac{\partial \phi_i}{\partial t} dV &\approx \int_V \sum_{i=1}^N \frac{\psi_i}{v_i} \phi_i \frac{dT(t)}{dt} dV \\ &= \frac{dT(t)}{dt} \int_V \sum_{i=1}^N \frac{\psi_i \phi_i}{v_i} dV \quad . \end{aligned} \quad (3.14)$$

Let

$$I = \int_V \sum_{i=1}^N \chi_i \psi_i \sum_{j=1}^N (\nu \Sigma_f)^j \phi_j dV \quad . \quad (3.15)$$

Now, we make the following definitions:

$$\Lambda = \frac{\int_V \sum_{i=1}^N \frac{\psi_i \phi_i}{\nu_i} dV}{I} \quad , \quad (3.16)$$

$$\beta_{\text{eff}} = \frac{\int_V \sum_{i=1}^N \chi_i^D \psi_i \sum_{j=1}^N \phi_j \sum_{m=1}^{ND} \beta_m (\nu \Sigma_f)_m^j dV}{I} \quad , \quad (3.17)$$

$$C_K = \frac{\int_V \sum_{i=1}^N \chi_i^D \psi_i C_K dV}{\Lambda I} \quad . \quad (3.18)$$

The next step is to substitute Equations (3.12), (3.13), (3.14), (3.15), (3.16), (3.17) and (3.18) into Equations (3.8) and (3.9). Equation (3.8) reduces to

$$\begin{aligned} \Lambda I \frac{dT(t)}{dt} = & \rho IT(t) - \beta_{\text{eff}} IT(t) + \Lambda I \sum_{K=1}^{NP} \lambda_K C_K(t) \\ & + \int_V \sum_{j=1}^N \phi_j \sum_{m=1}^{ND} \beta_m (\nu \Sigma_f)_m^j \left( \sum_{i=1}^N \chi_i^D \psi_i - \sum_{i=1}^N \chi_i \psi_i \right) dV \quad . \quad (3.19) \end{aligned}$$

Equation (3.9) reduces to

$$\Lambda I \frac{dC_K(t)}{dt} = \beta_{\text{eff}}^K IT(t) - \lambda_K C_K(t) \Lambda I \quad (3.20)$$

The last integral in Equation (3.19) is simply the difference in worth between the delayed neutrons and an equivalent number of prompt neutrons. Since the number of delayed neutrons is small, and the difference in their worths is also small, the product is a very small number and can be dropped for most purposes. Dividing Equations (3.19) and (3.20) by  $\Lambda I$ ,

$$\frac{dT(t)}{dt} = \frac{\rho - \beta_{\text{eff}}}{\Lambda} T(t) + \sum_{K=1}^{NP} \lambda_K C_K(t) \quad (3.21)$$

$$\frac{dC_K(t)}{dt} = \frac{\beta_{\text{eff}}^K}{\Lambda} T(t) - \lambda_K C_K(t) \quad (3.22)$$

The expressions for beta effective and neutron generation time in Equations (3.16) and (3.17), therefore, result in equations in the same form as the kinetics equations, Equations (3.10) and (3.11). There are several obvious similarities between the equations for beta effective and neutron generation time and the perturbation equation derived in the previous section. For example,  $I$  appears in all of the denominators, and the steady-state flux and adjoint flux appear in all expressions.

IV. GENERAL DESCRIPTION OF PERT-IV

Limits on Problem Complexity

PERT-IV is written in FORTRAN-IV for use on the UNIVAC 1107 or 1108. The maximum value of the input variables are as follows:

<u>Variable</u>	<u>Definition</u>	<u>Maximum Value</u>
IGM	No. of Energy Groups	26
IST	No. of Downscattering Terms	10
IM	No. of Radial Intervals	50
JM	No. of Axial Intervals	50
MCR	No. of Cross Sections from Cards	25
MTP	No. of Cross Sections from Tape	25
IZM	No. of Material Zones	35
MT	No. of Materials Including Mixes	35
MO1	No. of Mixture Specifications	45
NDELK	No. of Reactivity Traverses	20
NIBC	No. of Isotopes in $\beta_{eff}$ Calculation	20
NACT	No. of Activity Traverses	3

A 65K core and 4 scratch tapes are required for the above specifications. If a smaller core must be used, either reducing the number of radial and axial intervals or the number of groups is an easy way to decrease storage. Running time for a typical problem (5 groups, 20 reactivity traverses) is about 20 seconds on the 1108.

### Format of Input Fluxes and Adjoint Fluxes

The input flux and adjoint flux options are:

- Flux and adjoint from 1-D
- Flux and adjoint from 2-D
- Flux from 1-D, adjoint from 2-D
- Flux from 2-D, adjoint from 1-D
- Flux from 1-D, no adjoint
- Flux from 2-D, no adjoint.

When 1-D flux dumps are used, PERT-IV synthesizes these into 2-D arrays by assuming that the fluxes and adjoint fluxes are separable in the two dimensions over the whole reactor. For the last two options, the code assumes a flat adjoint distribution. Flux input can be taken directly from DTF-IV or 2DF, although other diffusion or transport theory codes may be used to provide fluxes and adjoint fluxes.

### Flux Normalization Options

Four options are available for normalizing the flux. These are:

- Center point flux
- Center point power density
- Total reactor power
- Denominator of perturbation equation (see Equation 2.21).

The adjoint flux is always normalized such that the fission source times the adjoint, summed over all groups, is unity at the center mesh point.

### Reactivity Coefficient Calculations

PERT-IV will calculate reactivity coefficient (in units of  $\delta k/k$  per kg) traverses for any specified row or column in R-Z, X-Y, or R- $\theta$  geometry. The contribution of each component of the reactivity coefficient (e.g., moderation, capture) is also given.

### Neutron Generation Time and Beta Effective Calculations

Using Equations (3.16) and (3.17), PERT-IV calculates the neutron generation time and the effective delayed neutron fraction. Additional input includes the neutron velocities for each group, the absolute number of neutrons per fission for each fissionable isotope ( $= \beta v$ ), and the delayed fission source. A single delayed fission source distribution is assumed for all delayed groups and isotopes.

### Activity Traverses

For any material and cross section (e.g.,  $\sigma_f$ ,  $\sigma_a$ ), the code will calculate and print an activity map. A full print gives the activity at each mesh point for each group; a partial print gives the total over all groups at each mesh point. Zone averaged activities are also given.

### Other Options

Minor options in PERT-IV include:

- Option to punch either flux or adjoint flux, or both.



- Full print option to print flux, adjoint flux and activity traverses for each group and an extended print out of cross sections.
- Option to read in a value for  $k_{\text{eff}}$  to be used in reactivity coefficient calculations--useful when  $k_{\text{eff}}$  as calculated by PERT-IV is significantly different from the transport theory value.

### Typical Results

A plot of the axial worth of Pu239 in a typical fast reactor as calculated by PERT-IV is shown in Figure 4.1. The graph also includes each component to the total reactivity coefficient. As one would expect, the magnitude of the contribution due to fissions, absorptions, and moderations is a maximum in the center of the core and decreases to the edge of the reactor. The leakage component, however, is zero at the center, reaches a maximum at the core-reflector interface, and then decreases to the reactor edge.

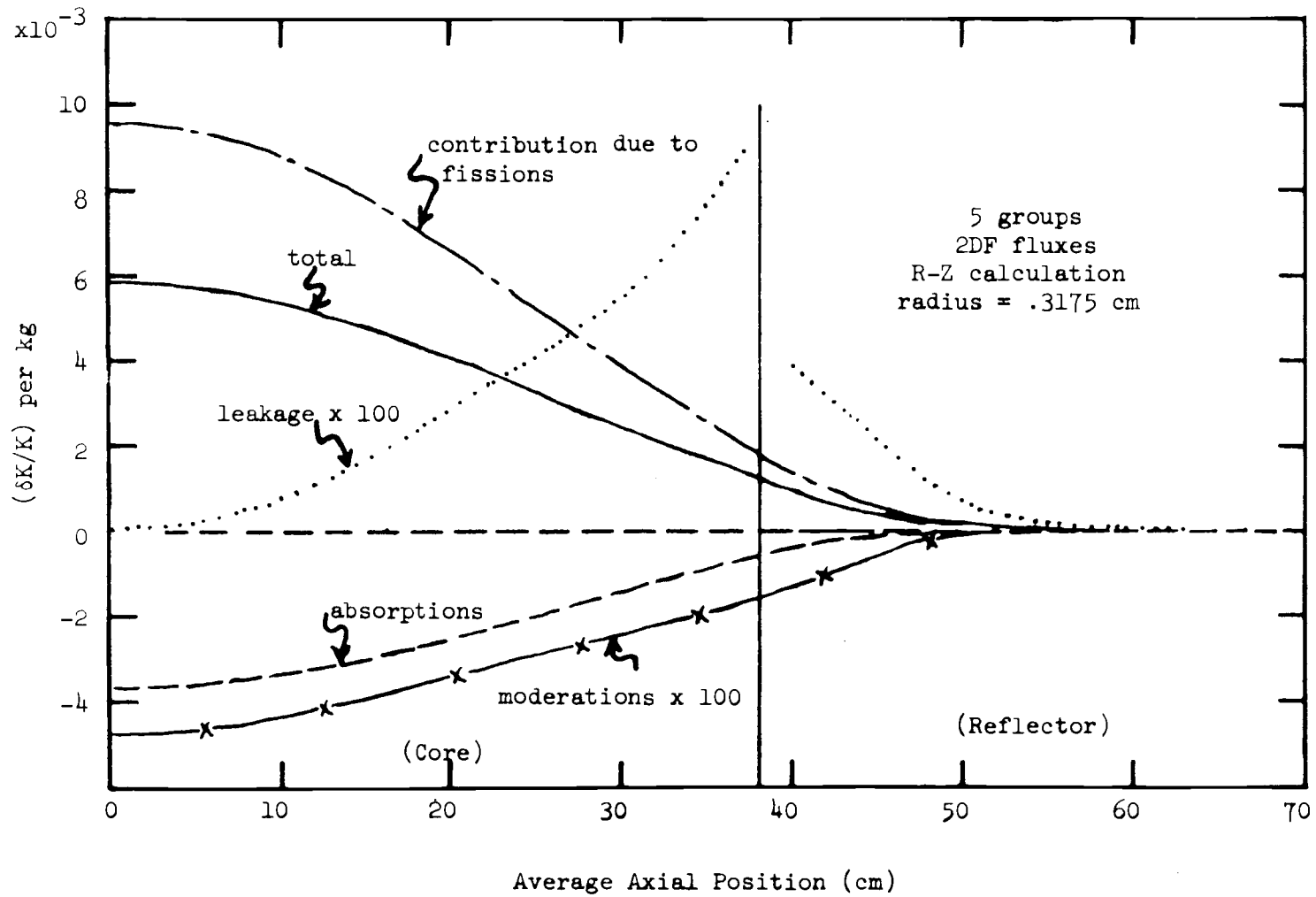


Figure 4.1. Axial worth curve of Pu-239 in ZPR-3 Assembly 48

V. COMPARISON BETWEEN DIFFUSION  
PERTURBATION THEORY AND TRANSPORT THEORY

Since the best way to assess diffusion perturbation theory is to compare results with a two-dimensional transport theory code, reactivity coefficients were computed using both PERT-IV and 2DF for a variety of materials and energy groups for a typical fast reactor core. Such a comparison is necessarily incomplete, however, because of the numerous items to check and the long time required for a 2-D transport theory calculation.

Calculations were performed using R-Z geometry for ZPR-III Assembly 48, (6, p. 57-64) a zero power fast critical assembly located at Idaho Falls. ZPR-III Assembly 48 was a 417 liter, plutonium fueled core, cylindrical in shape, and surrounded by a 30 cm thick reflector in both the axial and radial directions. The reactor dimensions and composition are given in Figure 5.1 and Table 5.1.

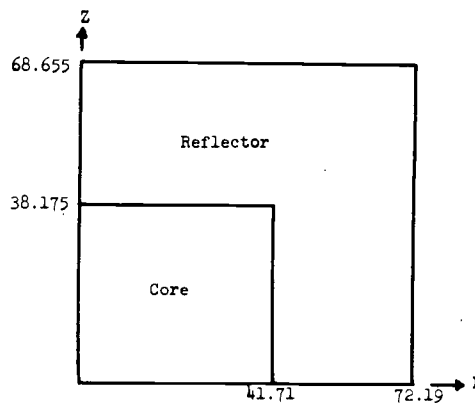


Figure 5.1. Reactor dimensions for ZPR-3 Assembly 48

Table 5.1

Reactor Composition for ZPR-III Assembly 48

(atoms-barn<sup>-1</sup>-cm<sup>-1</sup>)

<u>Material</u>	<u>Core Atom Densities</u>	<u>Reflector Atom Densities</u>
Pu239	.001654	.0
Pu240	.000106	.0
U235	.000016	.000084
U238	.007406	.039976
C	.020765	.0
Na	.006230	.0
Fe	.009899	.004453
Cr	.002658	.001196
Ni	.001308	.000588
Mo	.000206	.0
Al	.000110	.0

Cross section input consisted of a modified version of the Russian data compilation.<sup>(1)</sup> The data was punched using FCC,<sup>(9)</sup> a fundamental mode fast reactor code that will punch an n group Russian cross section set, where  $26 \geq n \geq 1$ .

The calculated reactivity coefficients in units of ( $\delta k/k$ ) per kg in the center of ZPR-3 Assembly 48 (R-Z geometry) are shown in Table 5.2. To negate the effect of sample size, most of the 2DF

Table 5.2

Calculated Reactivity Coefficients  
in the Center of ZPR-3 Assembly 48

Row	Number of Groups	Code	Order of SN Approximation	Pu239	Pu240	U235	Na	Fe	B10
1	1	2DF	S2	5.71 <sup>†</sup>	-	4.09	-	-	-
2	1	PERT 2DF fluxes	S2	5.487	.429	3.942	-.098	-.171	-96.61
3	1	2DF	S4	5.45	-	-	-	-	-
4	1	PERT 2DF fluxes	S4	5.338	.429	3.847	-.094	-.164	-
5	1	2DF	S6	5.43	-	-	-	-	-
6	1	PERT 2DF fluxes	S6	5.336	.429	3.845	-.094	-.164	-
7	2	2DF	S2	5.86	.47	4.19	-.126	-.183	-99.58
8	2	PERT 2DF fluxes	S2	5.587	.450	4.045	-.118	-.174	-96.14
9	2	2DF	S4	5.56	-	-	-	-	-
10	2	PERT 2DF fluxes	S4	5.423	.446	3.931	-.110	-.167	-92.60
11	2	2DB	D.T.	5.21	-	-	-	-	-
12	2	PERT 2DB fluxes	D.T.	5.22	-	-	-	-	-
13	5	2DF	S2	6.17	-	-	-	-	-
14	5	PERT 2DF fluxes	S2	5.867	.496	4.254	-.113	-.182	-99.16
15	5	PERT DTF fluxes	S4	5.688	.513	4.126	-.118	-.178	-93.86
16	26	PERT DTF fluxes	S4	5.885	.615	4.252	-.207	-.195	-84.42

<sup>†</sup> Units are ( $\delta k/k$ ) per kg x 10<sup>3</sup>

reactivity coefficient cases were run twice--with the sample in the test zone for both a positive and negative density and then the two averaged.

Rows 7 and 8 give a direct comparison between 2DF and PERT-IV calculations for several materials using two groups and the S2 option. The discrepancy between transport theory and diffusion perturbation theory is less than 4% for the fuels and the poison; for Na and Fe, however, the error is larger because their worths are the difference between two contributions (absorption and moderation) of the same magnitude. As an interesting sidelight, Los Alamos has written a one-dimensional transport perturbation theory code, DAC, compatible with DTF-IV, and report<sup>(5)</sup> differences in worths calculated by DAC and DTF-IV similar to those between PERT-IV and 2DF.

To determine whether the agreement is a function either of the number of energy groups or the order of the SN approximation, Pu239 worths were calculated using the S2, S4, and S6 options for 1 group, the S2 and S4 options for 2 groups, and the S2 option for 5 groups. A plot of this data can be seen in Figure 5.1. The percent error does not appear dependent upon the number of energy groups; however, the discrepancy decreases somewhat for higher orders of the SN approximation for both the one group and two group worth calculations. It is interesting to note that the 2DF value seems to be converging towards the PERT-IV value.

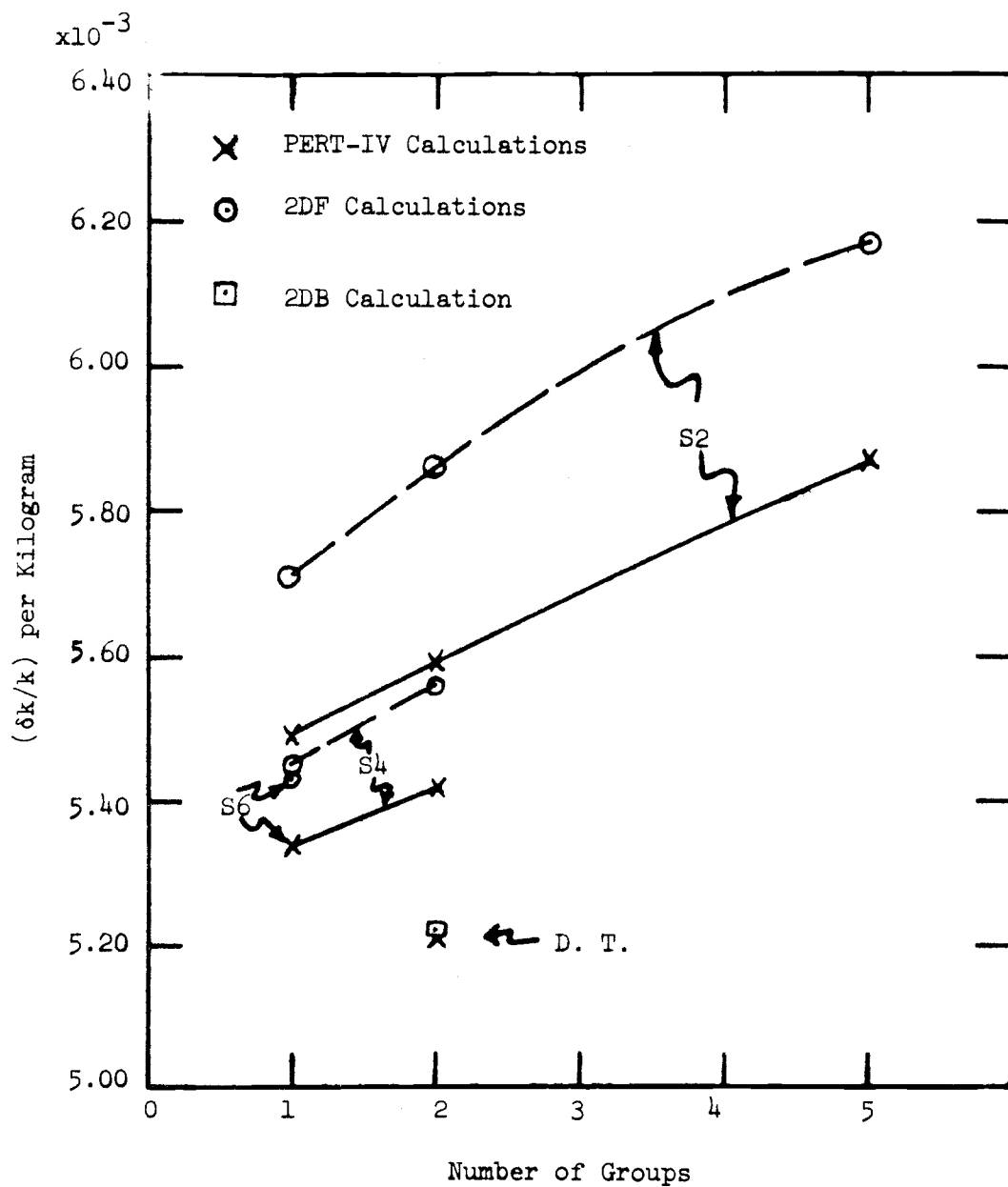


Figure 5.1. Comparison between PERT-IV, 2DF, and 2DB calculations in ZPR-3 Assembly 48

The worth of Pu239 was calculated using the two-dimensional diffusion theory program, 2DB, and compared with PERT-IV (using 2DB flux dumps). These results are shown in rows 11 and 12. As one would expect, since both codes are based on diffusion theory, the agreement is very good. Therefore, we conclude that inaccuracies in PERT-IV worths when compared to 2DF is because PERT-IV is based on the diffusion model.

The last two rows of Table 5.2 are PERT-IV calculations using one-dimensional DTF-IV fluxes. For this particular core there appears to be good agreement between PERT-IV calculations using DTF-IV fluxes and those using 2DF fluxes. It is not expected that this agreement will be as good for a multi-zoned reactor, however, since the assumption that the fluxes and adjoint fluxes are separable over the whole reactor will be less valid.

Recapitulating the above paragraphs, for the reactor studied, worths calculated using diffusion perturbation theory agree well with transport theory--especially fuel and poison worths. The agreement does not appear to be strongly dependent either on the number of groups or the order of the SN approximation in 2DF. If a two-dimensional diffusion theory code is used, the agreement is very good. Finally, using 1-D transport theory fluxes (radial and axial) in the diffusion perturbation code agrees fairly well with 2-D transport theory fluxes.



## VI. SUMMARY AND CONCLUSIONS

This thesis has described a two-dimensional, first order perturbation theory computer code based on the multigroup diffusion model that will compute reactivity coefficient traverses, the neutron generation time, and the effective delayed neutron fraction. The derivations of the perturbation equation and of the expressions for the neutron generation time and effective delayed neutron fraction as used in the code were given. Finally, some comparisons between reactivity coefficients calculated by diffusion perturbation theory and transport theory were shown.

Given the type of errors shown in Table 5.2, one must now ask if PERT-IV is a useful method for calculating reactivity coefficients. One criterion could be that the discrepancy between PERT-IV and 2DF worths should be less than the discrepancy between two 2DF calculations where the number of groups or the order of the SN approximation is varied. For the reactor studied in this report, this criterion is certainly met for Pu239 since there is over a 10% change from a 1 group to a 5 group 2DF calculation and a 5% change from S2 to S4, while the difference between PERT-IV and 2DF worths is less than 4%.

BIBLIOGRAPHY

1. Abagyan, L. P. et al. Group constants for nuclear reactor calculations. New York, Consultants Bureau, 1964. 151 p.
2. Astley, E. R. The conceptual evaluation of a fast flux test facility. 1967. 28 p. (U.S. Atomic Energy Commission. BNWL-SA-978)
3. Battat, M. E. and R. J. LaBouve. Comparison of multigroup cross-section sets used in reactor calculations. In: Proceedings of the international conference on fast critical experiments and their analysis, Argonne National Laboratory, Oct. 10-13, 1966. p. 130-134, (U.S. Atomic Energy Commission. ANL-7320)
4. Brooks, harvey. Perturbation methods in multigroup calculations. 1948. 13 p. (U.S. Atomic Energy Commission. KAPL-71)
5. Crawford, Bruce W. Multigroup kinetics for zoned nuclear reactors. Master's thesis. Berkeley, University of California, 1961. 58 numb. leaves.
6. Davey, W. G. Intercomparison of calculations for a dilute plutonium-fueled fast critical assembly (ZPR-3 Assembly 48). In: Proceedings of the international conference on fast critical experiments and their analysis, Argonne National Laboratory, Oct. 10-13, 1966. p. 57-64. (U.S. Atomic Energy Commission. ANL-7320)
7. Hardie, R. W. and W. W. Little, Jr. 1DX, a one-dimensional diffusion code for generating effective nuclear cross sections. 1968. (U.S. Atomic Energy Commission. To be published)
8. Lathrop, K. D. DTF-IV, a FORTRAN program for solving the multigroup transport equation with anisotropic scattering. 1965. 144 p. (U.S. Atomic Energy Commission. LA-3373)
9. Little, Jr., W. W. and R. W. Hardie. FCC - a fundamental mode code for fast reactor analysis. 1966. 127 p. (U.S. Atomic Energy Commission. BNWL-234)
10. Little, Jr., W. W. and R. W. Hardie. 2DB, a two-dimensional diffusion-burnup code for fast reactor analysis. 1967. 110 p. (U.S. Atomic Energy Commission. BNWL-640)

11. Shapiro, Martin. 2DF, a two-dimensional  $S_n$  code. Computer code received from Los Alamos Scientific Laboratory, 1965. (Unpublished)
12. Thalgott, F. W. et al. Fast critical experiments and their analysis. In: Proceedings of the Third International Conference on the Peaceful Uses of Atomic Energy, Geneva, 1964. Vol. 6. New York, United Nations, 1965. p. 124-133.
13. Wigner, Eugene P. Effect of small perturbations on pile period. 1945. 25 p. (U.S. Atomic Energy Commission. CP-3048)

A P P E N D I C E S

APPENDIX I: INPUT INSTRUCTIONS

The following two pages describe the input data for PERT-IV.

For DTF-IV fluxes and all subsequent data, the cards consist of six data fields of 12 columns each. The last nine columns contain the data associated with the particular field; columns 2-3 contain an integer, N, from 0 (or blank) to 99.

The first column of each field must contain:

- 0 - no effect (N=0),
- 1 - repeat associated data entry N times,
- 2 - do N linear interpolations between  
associated data entry and succeeding  
data entry,
- 3 - terminate reading of this array with  
previous data entry.

# PERT - IV

1	2	3	4	5	6	7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38	39	40	41	42	43	44	45	46	47	48	49	50	51	52	53	54	55	56	57	58	59	60	61	62	63	64	65	66	67	68	69	70	71	72	73	74	75	76	77	78	79	80	
IDENTIFICATION CARD																																																																																101121
<u>ND</u> * 1, Fluxes from 1-D * 2, Fluxes from 2-D * 3, Flux from 1-D, Adjacent from 2-D * 4, Flux from 2-D, Adjacent from 1-D * 5, Flux from 1-D, No Adjacent * 6, Flux from 2-D, No Adjacent		<u>IGM</u> No. of Energy Groups	<u>IST</u> No. of Downscattering Terms	<u>IFP</u> * 0, No Effect * 1, Punch Fluxes * 2, Punch Adjacent Fluxes	<u>IM</u> No. of Radial Intervals	<u>JM</u> No. of Axial Intervals	<u>MCR</u> No. of Materials From Cards	<u>MTP</u> No. of Materials From Tape	<u>I2M</u> No. of Material Zones	<u>MT</u> Total No. of Materials, Including Mixes	<u>MDI</u> No. of Mixture Specifications	<u>NPRT</u> * 0, Partial Print * 1, Full Print																																																																				
<u>NDELK</u> * 0, No Effect * n, n Reactivity Coefficient Calculations	<u>NIBC</u> * 0, No Effect * n, n Isotopes for $k_{eff}$ Calculation	<u>NACT</u> * 0, No Effect * n, n Activities	<u>NFP</u> Flux Normalization: * 1, Center Pt. Flux * 2, Center Pt. Power Density * 3, Total Power * 4, Denominator of Perturbation Formula	<u>IGE</u> Geometry: * 0, R-Z * 1, X-Y * 2, R- $\theta$																																																																												
<u>FLPD</u> Flux or Power (See NFP)	<u>VF</u> Volume Factor	<u>FER</u> Energy Released/Fission (MeV)																																																																														
<u>HOLDNMCR1</u> Name of 1st Isotope	<u>ATW(MCR)</u> Atomic Wt. for 1st Isotope	<u>AMCR (ID)</u> Miscellaneous Information for 1st Isotope (10A)																																																																														
Cross Section Data for 1st Isotope (OE I2.4)																																																																																C11TL, IGM, MT)
● ● ● (Repeat for Each Material)																																																																																
FLUX AND/OR ADJOINT FLUX: 2DF DATA (E12.4) OR DF DATA (E11, I2, ER-8)																																																																																PH11IM, JM1 AND ADJ1IM, JM1)
0.0	2ND RADIAL PT.	3RD RADIAL PT.	● ● ●	R111P1)																																																																												
0.0	2ND AXIAL PT.	3RD AXIAL PT.	● ● ●	Z111P1)																																																																												
ZONE NO. FOR 1st INTERVAL	ZONE NO. FOR 2ND INTERVAL	● ● ●	M011M, JM1)																																																																													
MATERIAL NO. FOR ZONE 1	MATERIAL NO. FOR ZONE 2	● ● ●	M211ZM:																																																																													
1	2	3	4	5	6	7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38	39	40	41	42	43	44	45	46	47	48	49	50	51	52	53	54	55	56	57	58	59	60	61	62	63	64	65	66	67	68	69	70	71	72	73	74	75	76	77	78	79	80	

	1	2	3	4	5	6	7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38	39	40	41	42	43	44	45	46	47	48	49	50	51	52	53	54	55	56	57	58	59	60	61	62	63	64	65	66	67	68	69	70	71	72	73	74	75	76	77	78	79	80
	FISSION SOURCE IN GROUP 1												FISSION SOURCE IN GROUP 2												• • •												CH111GM1																																											
	FIRST MIX NO.												• • •												SECOND MIX NO.												• • •												I01M01																															
	0												1ST MATERIAL IN MIX 1												2ND MATERIAL IN MIX 1												• • •												I11M01																															
	0												DENSITY FOR 1ST MATERIAL IN MIX 1												DENSITY FOR 2ND MATERIAL IN MIX 1												• • •												I21M01																															
IF(NIBC)≠ 0	MATERIAL NO. OF 1ST ISOTOPE FOR $\beta_{eff}$ CALCULATION												MATERIAL NO. OF 2ND ISOTOPE FOR $\beta_{eff}$ CALCULATION												• • •												NBETINIBC1																																											
	I FOR 1ST MATERIAL FOR $\beta_{eff}$ CALCULATION												I FOR 2ND MATERIAL FOR $\beta_{eff}$ CALCULATION												• • •												BETAINIBC1																																											
	DELAYED FISSION SOURCE IN GROUP 1												DELAYED FISSION SOURCE IN GROUP 2												• • •												CH1P1GM1																																											
	NEUTRON VELOCITY IN GROUP 1												NEUTRON VELOCITY IN GROUP 2												• • •												VEL11GM1																																											
IF(INDELK)≠ 0	MATERIAL NO. FOR 1ST $\beta k$ CALCULATION												MATERIAL NO. FOR 2ND $\beta k$ CALCULATION												• • •												MATDKINDELK1																																											
	ROW NO. FOR 1ST $\beta k$ CALCULATION												ROW NO. FOR 2ND $\beta k$ CALCULATION												• • •												NR0WINDELK1																																											
	COLUMN NO. FOR 1ST $\beta k$ CALCULATION												COLUMN NO. FOR 2ND $\beta k$ CALCULATION												• • •												NCOLINDELK1																																											
IF(INACT)≠ 0	MATERIAL NO. FOR 1ST ACTIVITY TRAVERSE												MATERIAL NO. FOR 2ND ACTIVITY TRAVERSE												• • •												MAINACT1																																											
	CROSS SECTION POSITION FOR 1ST ACTIVITY TRAVERSE												CROSS SECTION POSITION FOR 2ND ACTIVITY TRAVERSE												• • •												NXINACT1																																											

APPENDIX II: SAMPLE PROBLEM

The following pages show the input data (omitting input fluxes) and computer output for a two-group, three-region test problem in R-Z geometry.



PERT-IV TEST CASE		(2DF FLUX INPUT, 2 GROUPS, 3 ZONES)									
2	2	1	0	21	20	16	0	3	19	31	0
3	6	1	1	0							
	1.0		2.0		206.						
M2PU49239.05		2 GROUPS									
	.172436+01	.184872+01	.511503+01	.669364+01	.476589+01	.000000					
	.228419+01	.324006+01	.649503+01	.139176+02	.106775+02	.790403-01					
PU240 240.05		2 GROUPS									
	.697066-00	.973045-00	.210364+01	.654004+01	.549212+01	.000000					
	.205185-01	.158456+01	.577565-01	.139455+02	.123609+02	.748743-01					
M2U235235.04		2 GROUPS									
	.132033+01	.150965+01	.332484+01	.622874+01	.457201+01	.000000					
	.336017+01	.486972+01	.814961+01	.147169+02	.984715+01	.147083-00					
U238 238.05		2 GROUPS									
	.100100+00	.232887-00	.281437-00	.633569+01	.600524+01	.000000					
	.000000	.532188-00	.000000	.131583+02	.126261+02	.975608-01					
C 12.011		2 GROUPS									
	.000000	.833620-05	.000000	.263926+01	.245083+01	.000000					
	.000000	.456935-10	.000000	.448553+01	.448553+01	.188417-00					
NA 22.990		2 GROUPS									
	.000000	.713006-03	.000000	.309019+01	.300053+01	.000000					
	.000000	.423424-02	.000000	.498455+01	.498031+01	.889546-01					
M2 FE 55.847		2 GROUPS									
	.000000	.591775-02	.000000	.255761+01	.251745+01	.000000					
	.000000	.215431-01	.000000	.482144+01	.479990+01	.342399-01					
CR 51.996		2 GROUPS									
	.000000	.454490-02	.000000	.267339+01	.260924+01	.000000					
	.000000	.149873-01	.000000	.472111+01	.470613+01	.596010-01					
NI 58.71		2 GROUPS									
	.000000	.305271-01	.000000	.307692+01	.300395+01	.000000					
	.000000	.203676-01	.000000	.945337+01	.943300+01	.424393-01					
MO 95.94		2 GROUPS									
	.000000	.430943-01	.000000	.527468+01	.516105+01	.000000					
	.000000	.303487-00	.000000	.788962+01	.758613+01	.705369-01					
AL 26.981		2 GROUPS									
	.000000	.205447-02	.000000	.282519+01	.272632+01	.000000					
	.000000	.829689-02	.000000	.217056+01	.216227+01	.968106-01					
M2U235235.04		2 GROUPS									
	.134241+01	.156915+01	.333434+01	.679391+01	.505674+01	.000000					
	.211779+01	.283935+01	.515408+01	.123173+02	.947794+01	.168022-00					
U238 238.05		2 GROUPS									
	.372181-01	.189516-00	.105378+00	.694904+01	.664718+01	.000000					
	.000000	.404503-00	.000000	.122090+02	.118045+02	.112345+00					
M2 FE 55.847		2 GROUPS									
	.000000	.670574-02	.000000	.265143+01	.259971+01	.000000					
	.000000	.120269-01	.000000	.449904+01	.448701+01	.450092-01					
CR 51.996		2 GROUPS									

.000000	.483021-02	.000000	.275887+01	.267499+01	.000000
.000000	.923842-02	.000000	.441195+01	.440272+01	.790527-01
NI 58.71	2 GROUPS				
.000000	.182360-01	.000000	.338277+01	.330993+01	.000000
.000000	.169131-01	.000000	.750107+01	.748416+01	.546002-01
2 3 0.0	2 8 2.54	2 7 41.71	72.19 3		
2 3 0.0	2 7 2.54	2 7 38.175	68.655 3		
1 4	11 9	21 8	31 4	11 9	21 8 3
1 4	11 9	21 8	31 4	11 9	21 8 3
113	21 8	3113	2108	3113	21 8 3
113	21 8	3113	2108	3113	21 8 3
113	21 8	3113	2108	3184	3184 3
3					
	19	17	183		
.987	.013	3			
112	17106	18113	193		
	0	1	2	3	4 5
	6	7	8	9	10 11
	0	12	13	14	15 16
	0	1	2	3	4 5
	6	7	8	9	10 11
173					
0	.001654	.000106	.000016	.007406	.020765
.006230	.009899	.002658	.001308	.000206	.000110
0	.000084	.039976	.004453	.001196	.000588
0	.001654	.000106	.000016	.007406	.020765
.006230	.009899	.002658	.001308	.000206	.000110
1.0 -103					
1	2	3	4	12	13
3					
.0063	.0088	.0165	.0412	.0165	.0412
3					
1.0	0.03				
7.63085+8	1.16360+83				
1	1	63			
1	0	13			
0	5	03			
13					
13					

\* \* \* \* P E R T - I V \* \* \* \* \*

PERT-IV TEST CASE (2DF FLUX INPUT, 2 GROUPS, 3 ZONES)

ND	1/2/3/4/5/6/7=FLUXES FROM 1-D RUNS/FLUXES FROM 2-D RUN/FLUX FROM 1-D,ADJOINT FROM 2-D/FLUX FROM 2-D,ADJOINT FROM 1-D/FLUX FROM 1-D, NO ADJOINT/FLUX FROM 2-D,NO ADJOINT/2-D FLUXES FROM TAPE	2
IGM	NUMBER OF ENERGY GROUPS	2
IST	NUMBER OF DOWNSCATTERING TERMS	1
IFP	0/1/2=NO EFFECT/PUNCH FLUXES/PUNCH ADJOINT FLUXES	0
IM	NUMBER OF RADIAL INTERVALS	21
JM	NUMBER OF AXIAL INTERVALS	20
MCR	NUMBER OF MATERIALS FROM CARDS	16
MTP	NUMBER OF MATERIALS FROM TAPE	0
IZM	NUMBER OF MATERIAL ZONES	3
MT	TOTAL NUMBER OF MATERIALS INCLUDING MIXES	19
MO1	NUMBER OF MIXTURE SPECIFICATIONS	31
NPRT	PRINT OPTION [0/1=PARTIAL PRINT/FULL PRINT]	0
NDELK	0/N=NO EFFECT/DO K REACTIVITY COEFFICIENT CALCULATIONS	3
NIBC	0/N=NO EFFECT/USE N MATERIALS FOR BETA EFFECTIVE CALCULATION	6
NACT	0/N=NO EFFECT/DO N ACTIVITY TRAVERSES	1
NFP	FLUX OR POWER NORMALIZATION [1/2/3/4=CENTER PT. FLUX/CENTER PT. POWER/TOTAL POWER/DENOMINATOR OF PERT FORMULA]	1
IGE	GEOMETRY [0/1/2=R-Z/X-Y/R-THETA]	0
FLPO	FLUX(N/CM2-SEC) OR POWER(MWT)	1.0000+00
VF	VOLUME FACTOR	2.0000+00
FEF	ENERGY(MEV) RELEASED PER FISSION	2.0600+02
ZKEFF	KEFF FOR DELTA K CALC. [IF ZERO, USE KEFF CALCULATED BY CODE]	-0.0000

PERT-IV TEST CASE

(2DF FLUX INPUT, 2 GROUPS, 3 ZONES)

THE FOLLOWING NUCLIDES ARE HEAD-III

1	M2PU49	2 GROUPS
2	PU240	2 GROUPS
3	M2U235	2 GROUPS
4	U238	2 GROUPS
5	C	2 GROUPS
6	NA	2 GROUPS
7	M2 FE	2 GROUPS
8	CR	2 GROUPS
9	NI	2 GROUPS
10	MO	2 GROUPS
11	AL	2 GROUPS
12	M2U235	2 GROUPS
13	U238	2 GROUPS
14	M2 FE	2 GROUPS
15	CR	2 GROUPS
16	NI	2 GROUPS

PERT-IV TEST CASE

(2DF FLUX INPUT, 2 GROUPS, 3 ZONES)

INPUT FLUXES (PHR/PHZ/ADJR/ADJZ/PHI/ADJ=RADIAL FLUX/AXIAL FLUX/RADIAL ADJOINT FLUX/AXIAL ADJOINT FLUX/2-D FLUX/2-D ADJOINT FLUX)

PHI	840									
.48222-03	.48215-03	.48181-03	.48131-03	.47674-03	.46453-03	.44519-03	.41920-03	.38712-03	.34971-03	
.30787-03	.26261-03	.21373-03	.15665-03	.10533-03	.71757-04	.48494-04	.32370-04	.20945-04	.12615-04	
.58700-05	.48207-03	.48201-03	.48171-03	.48118-03	.47657-03	.46437-03	.44505-03	.41905-03	.38699-03	
.34959-03	.30777-03	.26253-03	.21366-03	.15660-03	.10529-03	.71732-04	.48477-04	.32363-04	.20934-04	
.12608-04	.58744-05	.48181-03	.48172-03	.48138-03	.48086-03	.47625-03	.46405-03	.44477-03	.41877-03	
.38672-03	.34937-03	.30756-03	.26235-03	.21351-03	.15649-03	.10522-03	.71680-04	.48449-04	.32341-04	
.20915-04	.12604-04	.58682-05	.48131-03	.48123-03	.48089-03	.48037-03	.47577-03	.46358-03	.44431-03	
.41936-03	.38632-03	.34901-03	.30726-03	.26208-03	.21330-03	.15634-03	.10511-03	.71611-04	.48404-04	
.32300-04	.20399-04	.12592-04	.58607-05	.47677-03	.47670-03	.47639-03	.47588-03	.47133-03	.45925-03	
.44014-03	.41444-03	.38271-03	.34572-03	.30436-03	.25962-03	.21129-03	.15486-03	.10412-03	.70937-04	
.47937-04	.32000-04	.20703-04	.12471-04	.58056-05	.46447-03	.46440-03	.46408-03	.46359-03	.45916-03	
.44739-03	.42877-03	.40371-03	.37279-03	.33676-03	.29645-03	.25285-03	.20578-03	.15083-03	.10140-03	
.69080-04	.46690-04	.31168-04	.20163-04	.12147-04	.56565-05	.44456-03	.44450-03	.44420-03	.44371-03	
.43948-03	.42821-03	.41037-03	.38639-03	.35677-03	.32224-03	.28366-03	.24193-03	.19687-03	.14428-03	
.97006-04	.66093-04	.44664-04	.29815-04	.19293-04	.11626-04	.54122-05	.41742-03	.41735-03	.41708-03	
.41663-03	.41263-03	.40204-03	.38528-03	.36273-03	.33493-03	.30247-03	.26617-03	.22698-03	.18470-03	
.13536-03	.90998-04	.61984-04	.41896-04	.27979-04	.18113-04	.10915-04	.50827-05	.38350-03	.38344-03	
.38318-03	.38277-03	.37910-03	.36935-03	.35394-03	.33317-03	.30759-03	.27777-03	.24437-03	.20829-03	
.16944-03	.12415-03	.83445-04	.56859-04	.38443-04	.25691-04	.16641-04	.10033-04	.46798-05	.34342-03	
.34337-03	.34314-03	.34276-03	.33948-03	.33074-03	.31690-03	.29829-03	.27529-03	.24852-03	.21854-03	
.18615-03	.15135-03	.11085-03	.74501-04	.50742-04	.34350-04	.23001-04	.14918-04	.90183-05	.42050-05	
.29787-03	.29782-03	.29762-03	.29730-03	.29444-03	.28686-03	.27484-03	.25866-03	.23867-03	.21533-03	
.18921-03	.16083-03	.13049-03	.95448-04	.64023-04	.43721-04	.29758-04	.19995-04	.13027-04	.78938-05	
.36874-05	.24605-03	.24600-03	.24584-03	.24557-03	.24322-03	.23694-03	.22701-03	.21362-03	.19709-03	
.17775-03	.15607-03	.13236-03	.10658-03	.77480-04	.52313-04	.36196-04	.24860-04	.16836-04	.11040-04	
.67154-05	.31495-05	.18350-03	.18346-03	.18334-03	.18314-03	.18138-03	.17670-03	.16928-03	.15928-03	
.14694-03	.13248-03	.11627-03	.98450-04	.78724-04	.58583-04	.41387-04	.29071-04	.20342-04	.13882-04	
.92031-05	.56163-05	.26504-05	.12639-03	.12636-03	.12627-03	.12614-03	.12493-03	.12171-03	.11660-03	
.10970-03	.10118-03	.91201-04	.80011-04	.67591-04	.54317-04	.42197-04	.31706-04	.22880-04	.16345-04	
.11320-04	.75492-05	.46643-05	.21899-05	.88048-04	.88025-04	.87964-04	.87864-04	.87027-04	.84790-04	
.81221-04	.76423-04	.70470-04	.63529-04	.55706-04	.47163-04	.38375-04	.30262-04	.23341-04	.17407-04	
.12667-04	.89861-05	.60253-05	.37616-05	.17789-05	.60744-04	.60727-04	.60681-04	.60613-04	.60042-04	
.58492-04	.56038-04	.52721-04	.48621-04	.43845-04	.38495-04	.32767-04	.26904-04	.21610-04	.17021-04	
.12923-04	.95290-05	.68654-05	.46995-05	.29230-05	.14119-05	.41347-04	.41334-04	.41305-04	.41266-04	
.40867-04	.39816-04	.38146-04	.35888-04	.33111-04	.29884-04	.26293-04	.22459-04	.18584-04	.15047-04	
.12033-04	.93601-05	.70041-05	.50583-05	.35128-05	.22122-05	.10564-05	.27270-04	.27263-04	.27247-04	
.27213-04	.26953-04	.26262-04	.25160-04	.23677-04	.21858-04	.19740-04	.17391-04	.14924-04	.12432-04	
.10177-04	.81899-05	.64100-05	.48988-05	.35856-05	.24835-05	.15853-05	.75639-06	.16746-04	.16741-04	
.16725-04	.16707-04	.16551-04	.16125-04	.15449-04	.14545-04	.13427-04	.12134-04	.10719-04	.92226-05	
.77128-05	.63370-05	.51624-05	.40823-05	.31106-05	.23068-05	.16179-05	.10283-05	.49905-06	.79327-05	

.79295-05	.79277-05	.79202-05	.78430-05	.76412-05	.73235-05	.68920-05	.63651-05	.57613-05	.50895-05
.43866-05	.36835-05	.30449-05	.24703-05	.19678-05	.15297-05	.11218-05	.78747-06	.50871-06	.24848-06
.23534-03	.23527-03	.23509-03	.23484-03	.23262-03	.22665-03	.21714-03	.20429-03	.18836-03	.16969-03
.14963-03	.12564-03	.10066-03	.72208-04	.47385-04	.31527-04	.20897-04	.13742-04	.87248-05	.50956-05
.21372-05	.23526-03	.23519-03	.23503-03	.23477-03	.23254-03	.22658-03	.21707-03	.20422-03	.18830-03
.16964-03	.14858-03	.12559-03	.10063-03	.72184-04	.47369-04	.31517-04	.20889-04	.13738-04	.87216-05
.50930-05	.21378-05	.23512-03	.23504-03	.23486-03	.23460-03	.23238-03	.22642-03	.21693-03	.20408-03
.18317-03	.16952-03	.14847-03	.12551-03	.10056-03	.72135-04	.47337-04	.31494-04	.20876-04	.13729-04
.87144-05	.50910-05	.21353-05	.23487-03	.23480-03	.23462-03	.23436-03	.23215-03	.22619-03	.21671-03
.20388-03	.18797-03	.16935-03	.14833-03	.12538-03	.10046-03	.72063-04	.47286-04	.31462-04	.20857-04
.13713-04	.87070-05	.50853-05	.21332-05	.23265-03	.23257-03	.23241-03	.23215-03	.22995-03	.22406-03
.21465-03	.20195-03	.18620-03	.16774-03	.14691-03	.12419-03	.99498-04	.71374-04	.46838-04	.31164-04
.20655-04	.13584-04	.86242-05	.50371-05	.21130-05	.22658-03	.22650-03	.22634-03	.22609-03	.22395-03
.21820-03	.20905-03	.19666-03	.18132-03	.16335-03	.14306-03	.12092-03	.96883-04	.69498-04	.45606-04
.30343-04	.20114-04	.13228-04	.83983-05	.49054-05	.20582-05	.21673-03	.21667-03	.21651-03	.21627-03
.21422-03	.20872-03	.19996-03	.18811-03	.17342-03	.15622-03	.13681-03	.11563-03	.92636-04	.66450-04
.43606-04	.29017-04	.19234-04	.12651-04	.80335-05	.46933-05	.19691-05	.20326-03	.20319-03	.20305-03
.20283-03	.20090-03	.19574-03	.18751-03	.17639-03	.16261-03	.14646-03	.12824-03	.10838-03	.86823-04
.62278-04	.40871-04	.27194-04	.18031-04	.11864-04	.75369-05	.44047-05	.18483-05	.18633-03	.18627-03
.18613-03	.18593-03	.18416-03	.17943-03	.17188-03	.16167-03	.14902-03	.13420-03	.11748-03	.99254-04
.79493-04	.57016-04	.37416-04	.24910-04	.16524-04	.10881-04	.69196-05	.40466-05	.17003-05	.16611-03
.16605-03	.16593-03	.16575-03	.16417-03	.15995-03	.15320-03	.14410-03	.13279-03	.11956-03	.10462-03
.88347-04	.70728-04	.50720-04	.33305-04	.22179-04	.14738-04	.97296-05	.61978-05	.36332-05	.15262-05
.14289-03	.14285-03	.14274-03	.14259-03	.14123-03	.13759-03	.13178-03	.12393-03	.11420-03	.10278-03
.89883-04	.75780-04	.60587-04	.43427-04	.28496-04	.19049-04	.12736-04	.84429-05	.54043-05	.31754-05
.13372-05	.11630-03	.11626-03	.11617-03	.11604-03	.11494-03	.11198-03	.10725-03	.10085-03	.92918-04
.83599-04	.73073-04	.61512-04	.48843-04	.34838-04	.23097-04	.15689-04	.10597-04	.70914-05	.45691-05
.26969-05	.11403-05	.84931-04	.84903-04	.84841-04	.84747-04	.83943-04	.81777-04	.78322-04	.73644-04
.67848-04	.61036-04	.53334-04	.44863-04	.35421-04	.25880-04	.18045-04	.12512-04	.86238-05	.58281-05
.37988-05	.22500-05	.95819-06	.57102-04	.57081-04	.57039-04	.56979-04	.56438-04	.54984-04	.52659-04
.49513-04	.45617-04	.41033-04	.35871-04	.30147-04	.24015-04	.18414-04	.13685-04	.97851-05	.68947-05
.47298-05	.31111-05	.18614-05	.79001-06	.38822-04	.38809-04	.38781-04	.38739-04	.38371-04	.37385-04
.35803-04	.33668-04	.31016-04	.27913-04	.24409-04	.20589-04	.16657-04	.13038-04	.99889-05	.74099-05
.53339-05	.37336-05	.24764-05	.14980-05	.63786-06	.26250-04	.26240-04	.26221-04	.26191-04	.25945-04
.25277-04	.24210-04	.22766-04	.20979-04	.18890-04	.16549-04	.14045-04	.11480-04	.91683-05	.71842-05
.54446-05	.40045-05	.28482-05	.19187-05	.11632-05	.50311-06	.17590-04	.17583-04	.17570-04	.17553-04
.17386-04	.16939-04	.16225-04	.15260-04	.14067-04	.12679-04	.1137-04	.94929-05	.78335-05	.63208-05
.50292-05	.38906-05	.29074-05	.20933-05	.14272-05	.87425-06	.37593-06	.11379-04	.11375-04	.11368-04
.11354-04	.11247-04	.10959-04	.10497-04	.98750-05	.91082-05	.82183-05	.72327-05	.61963-05	.51486-05
.42028-05	.33762-05	.26348-05	.20005-05	.14575-05	.99908-06	.61773-06	.26689-06	.67761-05	.67741-05
.67678-05	.67610-05	.66975-05	.65258-05	.62516-05	.58826-05	.54281-05	.49016-05	.43248-05	.37148-05
.31010-05	.25414-05	.20625-05	.16274-05	.12388-05	.91226-06	.63099-06	.38974-06	.17041-06	.28918-05
.2901-05	.28891-05	.28860-05	.28585-05	.27851-05	.26687-05	.25111-05	.23177-05	.20957-05	.18491-05
.15920-05	.13346-05	.11014-05	.89156-06	.70602-06	.54555-06	.39947-06	.27783-06	.17355-06	.74801-07
ADJ	840								
.30709-03	.30699-03	.30676-03	.30642-03	.30344-03	.29543-03	.28261-03	.26518-03	.24337-03	.21744-03
.18750-03	.15386-03	.11499-03	.70220-04	.35814-04	.18943-04	.99324-05	.53091-05	.27942-05	.14115-05
.53776-06	.30698-03	.30688-03	.30666-03	.30632-03	.30333-03	.29533-03	.28252-03	.26509-03	.24329-03
.21736-03	.18743-03	.15380-03	.11495-03	.79195-04	.35801-04	.18937-04	.99275-05	.53088-05	.27928-05

.14097-05	.53969-06	.30678-03	.30668-03	.31644-03	.30610-03	.30312-03	.29512-03	.28232-03	.26490-03
.24311-03	.21721-03	.18730-03	.15370-03	.11487-03	.70146-04	.35777-04	.18921-04	.99222-05	.53053-05
.27388-05	.14129-05	.53759-06	.30646-03	.36636-03	.30613-03	.30578-03	.30281-03	.29481-03	.28203-03
.26463-03	.24286-03	.21699-03	.16711-03	.15353-03	.11475-03	.70076-04	.35735-04	.18902-04	.99146-05
.52956-05	.27083-05	.14084-05	.53713-06	.36347-03	.30338-03	.30315-03	.30281-03	.29986-03	.29195-03
.27928-03	.26205-03	.24050-03	.21487-03	.18528-03	.15203-03	.11362-03	.69380-04	.35383-04	.18715-04
.98096-05	.52443-05	.27592-05	.13937-05	.53166-06	.29530-03	.29520-03	.29498-03	.29465-03	.29178-03
.28408-03	.27175-03	.25498-03	.23400-03	.20906-03	.16027-03	.14791-03	.11054-03	.67487-04	.34406-04
.18191-04	.95357-05	.59959-05	.26804-05	.13539-05	.51681-06	.28197-03	.28188-03	.28167-03	.28135-03
.27361-03	.27126-03	.25948-03	.24346-03	.22342-03	.19959-03	.17209-03	.14119-03	.10550-03	.64386-04
.32008-04	.17342-04	.90829-05	.48528-05	.25523-05	.12895-05	.49168-06	.26354-03	.26345-03	.26326-03
.26296-03	.26039-03	.25352-03	.24250-03	.22752-03	.20878-03	.18650-03	.16078-03	.13189-03	.98525-04
.60098-04	.30594-04	.16151-04	.84561-05	.45166-05	.23755-05	.12001-05	.45798-06	.24000-03	.23993-03
.23975-03	.23948-03	.23714-03	.23087-03	.22083-03	.20717-03	.19009-03	.16978-03	.14634-03	.12001-03
.89599-04	.54501-04	.14633-04	.14633-04	.76521-05	.40884-05	.21518-05	.10883-05	.41641-06	.21117-03
.21110-03	.21094-03	.21070-03	.20864-03	.20312-03	.19427-03	.18225-03	.16719-03	.14929-03	.12862-03
.10543-03	.78635-04	.47815-04	.24241-04	.12757-04	.66751-05	.35772-05	.18867-05	.95906-06	.36580-06
.17589-03	.17683-03	.17670-03	.17650-03	.17477-03	.17614-03	.16272-03	.15263-03	.13999-03	.12496-03
.10760-03	.88061-04	.65534-04	.39661-04	.19970-04	.10547-04	.55629-05	.29988-05	.15960-05	.81374-06
.31265-06	.13470-03	.13466-03	.13455-03	.13440-03	.13309-03	.12955-03	.12389-03	.11617-03	.10652-03
.95022-04	.81724-04	.66718-04	.49244-04	.29563-04	.15074-04	.81656-05	.43711-05	.23979-05	.12901-05
.66486-06	.25789-06	.84091-04	.84063-04	.84000-04	.83903-04	.80862-04	.77306-04	.72454-04	.72454-04
.66383-04	.59123-04	.50736-04	.41207-04	.36144-04	.18975-04	.10649-04	.59261-05	.33191-05	.18350-05
.10238-05	.52310-06	.21141-06	.44358-04	.44341-04	.44307-04	.44261-04	.43821-04	.42642-04	.40748-04
.38162-04	.34920-04	.31029-04	.26555-04	.21415-04	.15834-04	.10928-04	.70218-05	.41373-05	.24267-05
.13831-05	.78312-06	.41549-06	.16029-06	.24161-04	.24153-04	.24136-04	.24107-04	.23867-04	.23222-04
.22178-04	.20761-04	.18970-04	.16841-04	.14374-04	.11629-04	.88106-05	.62500-05	.42492-05	.27330-05
.16716-05	.10076-05	.57308-06	.31701-06	.12240-06	.13018-04	.13014-04	.13002-04	.12985-04	.12859-04
.12506-04	.11942-04	.11167-04	.10197-04	.90412-05	.77209-05	.62958-05	.48415-05	.35891-05	.25577-05
.17154-05	.11963-05	.69200-06	.41526-06	.22405-06	.96053-07	.71149-05	.71117-05	.71059-05	.71011-05
.70267-05	.68337-05	.65224-05	.60971-05	.55653-05	.49360-05	.42283-05	.34689-05	.27142-05	.20319-05
.14938-05	.10605-05	.70924-06	.45541-06	.28359-06	.15971-06	.63433-07	.38252-05	.38240-05	.38227-05
.38156-05	.37777-05	.36734-05	.35049-05	.32757-05	.29901-05	.26541-05	.22789-05	.18869-05	.14936-05
.11575-05	.86468-06	.61786-06	.43655-06	.29038-06	.18080-06	.10629-06	.43104-07	.19708-05	.19704-05
.19666-05	.19657-05	.19459-05	.18916-05	.18047-05	.16871-05	.15400-05	.13687-05	.11813-05	.98178-06
.78529-06	.60552-06	.46906-06	.34895-06	.24110-06	.16781-06	.10872-06	.61522-07	.27396-07	.76526-06
.76404-06	.76530-06	.76411-06	.75586-06	.73474-06	.70132-06	.65498-06	.59840-06	.53324-06	.45915-06
.38415-06	.31042-06	.24822-06	.18486-06	.13795-06	.10504-06	.67868-07	.45217-07	.27882-07	.10280-07
.36080-03	.36075-03	.36050-03	.36013-03	.35674-03	.34765-03	.33328-03	.31399-03	.29023-03	.26262-03
.23191-03	.19897-03	.16420-03	.12584-03	.86060-04	.59774-04	.40995-04	.27645-04	.18014-04	.10899-04
.50338-05	.36070-03	.36065-03	.36043-03	.36004-03	.35661-03	.34754-03	.33318-03	.31388-03	.29014-03
.26254-03	.23183-03	.19890-03	.16415-03	.12380-03	.86030-04	.59754-04	.40980-04	.27639-04	.18006-04
.10894-04	.50864-05	.36051-03	.36045-03	.36019-03	.35980-03	.35637-03	.34730-03	.33297-03	.31367-03
.23994-03	.26237-03	.23167-03	.19877-03	.16404-03	.12372-03	.85972-04	.59712-04	.40955-04	.27620-04
.17990-04	.19890-04	.50813-05	.36014-03	.36008-03	.35983-03	.35944-03	.35601-03	.34695-03	.33263-03
.31337-03	.28964-03	.26210-03	.23145-03	.19856-03	.16387-03	.12359-03	.85882-04	.59653-04	.40917-04
.27587-04	.17975-04	.19879-04	.50751-05	.35670-03	.35670-03	.35609-03	.35271-03	.35271-03	.34372-03
.32952-03	.31045-03	.28695-03	.25964-03	.22928-03	.19671-03	.16233-03	.12243-03	.85077-04	.59093-04
.45524-04	.27330-04	.17807-04	.10775-04	.50276-05	.34762-03	.34756-03	.34732-03	.34696-03	.34366-03

.33490-03	.32106-03	.30246-03	.27956-03	.25296-03	.22335-03	.19161-03	.15813-03	.11925-03	.82865-04
.57551-04	.39472-04	.26620-04	.17343-04	.10495-04	.48984-05	.33283-03	.33278-03	.33256-03	.33220-03
.32305-03	.32066-03	.30740-03	.28959-03	.26764-03	.24213-03	.21379-03	.18339-03	.15132-03	.11411-03
.79287-04	.55069-04	.37764-04	.25467-04	.16596-04	.10046-04	.46872-05	.31272-03	.31267-03	.31247-03
.31214-03	.30915-03	.30127-03	.28880-03	.27204-03	.25143-03	.22743-03	.20073-03	.17216-03	.14205-03
.10710-03	.74425-04	.51663-04	.35433-04	.23903-04	.15583-04	.94332-05	.44023-05	.28768-03	.28764-03
.23745-03	.28714-03	.28440-03	.27713-03	.26565-03	.25019-03	.23119-03	.20913-03	.18452-03	.15817-03
.13044-03	.98327-04	.68280-04	.47418-04	.32527-04	.21957-04	.14322-04	.86733-05	.40544-05	.25827-03
.25323-03	.25306-03	.25778-03	.25532-03	.24378-03	.23845-03	.22456-03	.20742-03	.18756-03	.16542-03
.14169-03	.11677-03	.87957-04	.61050-04	.42370-04	.29095-04	.19675-04	.12847-04	.77992-05	.36446-05
.22522-03	.22519-03	.22504-03	.22480-03	.22264-03	.21694-03	.20791-03	.19577-03	.18079-03	.16335-03
.14394-03	.12299-03	.10112-03	.76041-04	.52640-04	.36597-04	.25250-04	.17124-04	.11228-04	.68315-05
.31975-05	.18859-03	.13856-03	.18843-03	.18823-03	.18643-03	.18165-03	.17408-03	.16389-03	.15132-03
.13666-03	.12031-03	.19253-03	.83033-04	.62269-04	.43255-04	.30410-04	.21148-04	.14443-04	.95272-05
.58166-05	.27331-05	.14472-03	.14469-03	.14459-03	.14444-03	.14306-03	.13939-03	.13357-03	.12573-03
.11606-03	.10477-03	.92152-04	.78375-04	.63246-04	.47811-04	.34447-04	.24555-04	.17349-04	.11938-04
.79308-05	.48715-05	.23012-05	.10303-03	.10300-03	.10292-03	.10282-03	.10184-03	.99224-04	.95071-04
.89477-04	.82575-04	.74502-04	.65475-04	.55513-04	.44877-04	.35109-04	.26636-04	.19441-04	.13990-04
.97551-05	.65360-05	.40476-05	.19055-05	.73181-04	.73162-04	.73111-04	.73029-04	.72335-04	.70482-04
.67526-04	.63552-04	.58629-04	.52894-04	.46449-04	.39432-04	.32212-04	.25545-04	.19829-04	.14891-04
.10898-04	.77594-05	.52285-05	.32686-05	.15486-05	.51236-04	.51221-04	.51183-04	.51125-04	.50644-04
.49341-04	.47276-04	.44488-04	.41043-04	.37034-04	.32555-04	.27766-04	.22861-04	.18417-04	.14563-04
.11117-04	.82392-05	.59505-05	.40820-05	.25472-05	.12282-05	.35241-04	.35231-04	.35205-04	.35171-04
.34833-04	.33939-04	.32519-04	.30000-04	.28240-04	.25500-04	.22458-04	.19211-04	.15925-04	.12929-04
.10363-04	.80800-05	.60707-05	.43999-05	.30579-05	.19290-05	.92250-06	.23413-04	.23407-04	.23393-04
.23365-04	.23142-04	.22550-04	.21605-04	.20335-04	.18776-04	.16965-04	.14957-04	.12848-04	.10718-04
.37854-05	.70864-05	.55599-05	.42543-05	.31213-05	.21672-05	.13834-05	.66152-06	.14445-04	.14441-04
.14427-04	.14412-04	.14277-04	.13910-04	.13328-04	.12550-04	.11588-04	.10475-04	.92590-05	.79730-05
.66745-05	.54922-05	.44774-05	.35460-05	.27098-05	.20112-05	.14119-05	.89863-06	.43591-06	.68594-05
.63567-05	.68544-05	.68479-05	.67814-05	.66073-05	.63329-05	.59605-05	.55057-05	.49852-05	.44061-05
.38000-05	.31937-05	.26419-05	.21481-05	.17122-05	.13305-05	.97951-06	.68853-06	.44430-06	.21692-06

MESH BOUNDARIES (R1/Z1=RADIAL POINTS/AXIAL POINTS)

R1	22								
-.00000	.63500+00	.12700+01	.19050+01	.25400+01	.68922+01	.11244+02	.15597+02	.19949+02	.24301+02
.28553+02	.33026+02	.37358+02	.41710+02	.45520+02	.49330+02	.53140+02	.56950+02	.60760+02	.64570+02
.68380+02	.72190+02								
Z1	21								
-.00000	.63500+00	.12700+01	.19050+01	.25400+01	.69944+01	.11449+02	.15903+02	.20358+02	.24812+02
.29266+02	.33721+02	.38175+02	.41985+02	.45795+02	.49605+02	.53415+02	.57225+02	.61035+02	.64845+02
.68555+02									

ZONE NUMBERS BY MESH INTERVAL

N0	420								
1	1	1	1	2	2	2	2	2	2
2	2	2	3	3	3	3	3	3	3
3	1	1	1	1	2	2	2	2	2
2	2	2	2	3	3	3	3	3	3
3	3	1	1	1	1	2	2	2	2





17	17	18	18	18	18	18	18	19	19
19	19	19	19	19	19	19	19	19	19
19									
11	31								
0	1	2	3	4	5	6	7	8	9
10	11	0	12	13	14	15	16	0	1
2	3	4	5	6	7	8	9	10	11
17									
12	31								
.00000	.16540-02	.10600-03	.16000-04	.74060-02	.20765-01	.62300-02	.98990-02	.26580-02	.13080-02
.20000-03	.11000-03	.00000	.84000-04	.39976-01	.44530-02	.11960-02	.58800-03	.00000	.16540-02
.10600-03	.16000-04	.74060-02	.20765-01	.62300-02	.98990-02	.26580-02	.13080-02	.20600-03	.11000-03
.10000-09									

MATERIAL NUMBERS FOR BETA CALCULATION

NBET	6						
1	2	3	4	12	13		
BETA	6						
.63000-02	.88000-02	.16500-01	.41200-01	.16500-01	.41200-01		

DELAYED FISSION SPECTRUM

CHIP	2
.10000+01	.00000

NEUTRON VELOCITY

VEL	2
.76309+09	.11636+09

PERTURBATION CALCULATION INFORMATION (MATDK/NROW/NCOL=MATERIAL NUMBER/ROW/COLUMN TO BE PERTURBED)

MATDK	3		
1	1	6	
NROW	3		
1	0	1	
NCOL	3		
0	5	0	

ACTIVITY TRAVERSE INFORMATION (MA/NX=MATERIAL NUMBER/CROSS SECTION POSITION)

MA	1
1	
NX	1
1	

PERT-IV TEST CASE

(2DF FLUX INPUT, 2 GROUPS, 3 ZONES)

```
3 3 3 3 3 3 3 3 3 3 3 3 3 3 3 3 3 3
3 3 3 3 3 3 3 3 3 3 3 3 3 3 3 3 3 3
3 3 3 3 3 3 3 3 3 3 3 3 3 3 3 3 3 3
3 3 3 3 3 3 3 3 3 3 3 3 3 3 3 3 3 3
3 3 3 3 3 3 3 3 3 3 3 3 3 3 3 3 3 3
3 3 3 3 3 3 3 3 3 3 3 3 3 3 3 3 3 3
3 3 3 3 3 3 3 3 3 3 3 3 3 3 3 3 3 3
3 3 3 3 3 3 3 3 3 3 3 3 3 3 3 3 3 3
2 2 2 2 2 2 2 2 2 2 2 2 2 2 2 2 2 2
2 2 2 2 2 2 2 2 2 2 2 2 2 2 2 2 2 2
2 2 2 2 2 2 2 2 2 2 2 2 2 2 2 2 2 2
2 2 2 2 2 2 2 2 2 2 2 2 2 2 2 2 2 2
2 2 2 2 2 2 2 2 2 2 2 2 2 2 2 2 2 2
2 2 2 2 2 2 2 2 2 2 2 2 2 2 2 2 2 2
2 2 2 2 2 2 2 2 2 2 2 2 2 2 2 2 2 2
2 2 2 2 2 2 2 2 2 2 2 2 2 2 2 2 2 2
2 2 2 2 2 2 2 2 2 2 2 2 2 2 2 2 2 2
2 2 2 2 2 2 2 2 2 2 2 2 2 2 2 2 2 2
1 1 1 1 2 2 2 2 2 2 2 2 2 2 2 2 2 2
1 1 1 1 2 2 2 2 2 2 2 2 2 2 2 2 2 2
1 1 1 1 2 2 2 2 2 2 2 2 2 2 2 2 2 2
1 1 1 1 2 2 2 2 2 2 2 2 2 2 2 2 2 2
1 1 1 1 2 2 2 2 2 2 2 2 2 2 2 2 2 2
```

A  
X  
I  
A  
L

RADIAL

PERT-IV TEST CASE

(2DF FLUX INPUT, 2 GROUPS, 3 ZONES)

14181318191818181818181818181818181818181818  
13131818181818181818181818181818181818181818  
13181313131818181818181818181818181818181818  
13181818181818181818181818181818181818181818  
13181818181818181818181818181818181818181818  
13181818181818181818181818181818181818181818  
13181818181818181818181818181818181818181818  
18181818181818181818181818181818181818181818  
17171717171717171717171717171717181818181818  
17171717171717171717171717171717181818181818  
17171717171717171717171717171717181818181818  
17171717171717171717171717171717181818181818  
17171717171717171717171717171717181818181818  
17171717171717171717171717171717181818181818  
17171717171717171717171717171717181818181818  
19191919171717171717171717171718181818181818  
19191919171717171717171717171718181818181818  
19191919171717171717171717171718181818181818  
19191919171717171717171717171718181818181818

A  
X  
I  
A  
L  
  
RADIAL

PERT-IV TEST CASE

(2DF FLUX INPUT, 2 GROUPS, 3 ZONES)

CROSS-SECTION EDIT

GROUP 1 CROSS-SECTIONS

MAT 1	.172+01	.185+01	.512+01	.669+01	.477+01	.000
MAT 2	.697-00	.973-00	.210+01	.654+01	.549+01	.000
MAT 3	.132+01	.151+01	.532+01	.623+01	.457+01	.000
MAT 4	.100+00	.233-00	.281-00	.634+01	.601+01	.000
MAT 5	.000	.834-05	.000	.264+01	.245+01	.000
MAT 6	.000	.713-03	.000	.309+01	.300+01	.000
MAT 7	.000	.592-02	.000	.256+01	.252+01	.000
MAT 8	.000	.454-02	.000	.267+01	.261+01	.000
MAT 9	.000	.305-01	.000	.308+01	.300+01	.000
MAT 10	.000	.431-01	.000	.527+01	.516+01	.000
MAT 11	.000	.205-02	.000	.283+01	.273+01	.000
MAT 12	.134+01	.157+01	.533+01	.679+01	.506+01	.000
MAT 13	.372-01	.190-00	.105+00	.695+01	.665+01	.000
MAT 14	.000	.671-02	.000	.265+01	.260+01	.000
MAT 15	.000	.483-02	.000	.276+01	.267+01	.000
MAT 16	.000	.182-01	.000	.338+01	.331+01	.000
MAT 17	.369-02	.503-02	.108-01	.171-00	.160-00	.000
MAT 18	.160-02	.775-02	.449-02	.295-00	.283-00	.000
MAT 19	.369-02	.503-02	.108-01	.171-00	.160-00	.000

GROUP 2 CROSS-SECTIONS

MAT 1	.228+01	.324+01	.050+01	.139+02	.107+02	.790-01
MAT 2	.205-01	.158+01	.578-01	.139+02	.124+02	.749-01
MAT 3	.336+01	.487+01	.015+01	.147+02	.085+01	.147-00
MAT 4	.000	.532-00	.000	.132+02	.126+02	.976-01
MAT 5	.000	.457-10	.000	.449+01	.449+01	.188-00
MAT 6	.000	.423-02	.000	.498+01	.498+01	.890-01
MAT 7	.000	.215-01	.000	.482+01	.480+01	.342-01
MAT 8	.000	.150-01	.000	.472+01	.471+01	.596-01
MAT 9	.000	.204-01	.000	.945+01	.943+01	.424-01
MAT 10	.000	.303-00	.000	.789+01	.759+01	.705-01
MAT 11	.000	.830-02	.000	.217+01	.216+01	.968-01
MAT 12	.212+01	.284+01	.515+01	.123+02	.948+01	.168-00
MAT 13	.000	.405-00	.000	.122+02	.118+02	.112+00
MAT 14	.000	.120-01	.000	.459+01	.449+01	.450-01
MAT 15	.000	.924-02	.000	.441+01	.440+01	.791-01

MAT 16	.000	.169-01	.000	.750+01	.748+01	.546-01
MAT 17	.383-02	.992-02	.109-01	.321-00	.311-00	.591-02
MAT 18	.173-03	.165-01	.433-03	.510-00	.502-00	.483-02
MAT 19	.333-02	.992-02	.109-01	.321-00	.311-00	.591-02

PERT-IV TEST CASE

(2DF FLUX INPUT, 2 GROUPS, 3 ZONES)

MIXTURE NUMBER	MIX COMMAND	MATERIAL	ATOMIC DENSITY
1	17	0	.00000000
2	17	1	.16540000-02
3	17	2	.10600000-03
4	17	3	.16000000-04
5	17	4	.74060000-02
6	17	5	.20765000-01
7	17	6	.62300000-02
8	17	7	.98989999-02
9	17	8	.26580000-02
10	17	9	.13080000-02
11	17	10	.20600000-03
12	17	11	.11000000-03
13	18	0	.00000000
14	18	12	.83999999-04
15	18	13	.39976000-01
16	18	14	.44530000-02
17	18	15	.11960000-02
18	18	16	.58799999-03
19	19	0	.00000000
20	19	1	.16540000-02
21	19	2	.10600000-03
22	19	3	.16000000-04
23	19	4	.74060000-02
24	19	5	.20765000-01
25	19	6	.62300000-02
26	19	7	.98989999-02
27	19	8	.26580000-02
28	19	9	.13080000-02
29	19	10	.20600000-03
30	19	11	.11000000-03
31	19	17	.99999999-10

PERT-IV TEST CASE

(2DF FLUX INPUT, 2 GROUPS, 3 ZONES)

	RADII	AVG RADII	AXII	AVG AXII
1	-.0000	.3175	-.0000	.3175
2	.6350	.9525	.6350	.9525
3	1.2700	1.5875	1.2700	1.5875
4	1.9050	2.2225	1.9050	2.2225
5	2.5400	4.7161	2.5400	4.7672
6	6.8922	9.0683	6.9944	9.2216
7	11.2444	13.4206	11.4487	13.6759
8	15.5967	17.7728	15.9031	18.1303
9	19.9489	22.1250	20.3575	22.5847
10	24.3011	26.4772	24.8119	27.0391
11	28.6533	30.8294	29.2662	31.4934
12	33.0056	35.1817	33.7206	35.9478
13	37.3578	39.5339	38.1750	40.4020
14	41.7100	43.8861	41.9850	43.8900
15	45.5200	47.4250	45.7950	47.7000
16	49.3300	51.2350	49.6050	51.5100
17	53.1400	55.0450	53.4150	55.3200
18	56.9500	58.8550	57.2250	59.1300
19	60.7600	62.6650	61.0350	62.9400
20	64.5700	66.4750	64.8450	66.7500
21	68.3800	70.2850	68.6550	.0000
22	72.1900	.0000	.0000	.0000



PERT-IV TEST CASE

(2DF FLUX INPUT, 2 GROUPS, 3 ZONES)

MATERIAL INVENTORY (KILOGRAMS) FOR EACH ZONE

MATERIAL	ATOMIC WT.	ZONE 1 .10 LITERS	ZONE 2 417.19 LITERS	ZONE 3 1830.76 LITERS
1 M2PU49	239.050	.068	273.869	.000
2 PU240	240.050	.004	17.625	.000
3 M2U235	235.040	.001	2.605	.000
4 U238	238.050	.001	1221.155	.000
5 C	12.011	.043	172.755	.000
6 NA	22.990	.024	99.208	.000
7 M2 FE	55.847	.095	392.922	.000
8 CR	51.996	.024	95.729	.000
9 NI	58.710	.013	53.191	.000
10 MO	95.940	.003	13.689	.000
11 AL	26.981	.001	2.056	.000
12 M2U235	235.040	.000	.000	60.012
13 U238	238.050	.000	.000	28925.792
14 M2 FE	55.847	.000	.000	755.910
15 CR	51.996	.000	.000	189.025
16 NI	58.710	.000	.000	104.932

PERT-IV TEST CASE

(2DF FLUX INPUT, 2 GROUPS, 3 ZONES)

CENTER POINT FLUX (N/CM2-SEC)	-	-	-	1.00000+00
CENTER POINT POWER (MWT/LITER)	-	-	-	1.23298-16
TOTAL POWER (MWT)	-	-	-	3.22585-14
DENOMINATOR OF PERT. FORMULA	-	-	-	1.56242+03
SOURCE RATE	-	-	-	2.82202+03
ABSORPTION RATE	-	-	-	2.64820+03
LEAKAGE RATE (TOTAL)	-	-	-	1.69694+02
RADIAL LEAKAGE	-	-	-	1.18695+02
AXIAL LEAKAGE	-	-	-	4.19992+01
KEFF	-	-	-	1.00467

PERT-IV TEST CASE

(2DF FLUX INPUT, 2 GROUPS, 3 ZONES)

TOTAL FLUX

	1	2	3	4	5
1	.100000+01	.9997979-00	.9990830-00	.9980266-00	.9885766-00
2	.9996795-00	.9994885-00	.9988517-00	.9977479-00	.9882198-00
3	.9991206-00	.9988865-00	.9981660-00	.9970762-00	.9875564-00
4	.9980810-00	.9978705-00	.9971487-00	.9960616-00	.9865670-00
5	.9886532-00	.9884525-00	.9877933-00	.9867203-00	.9773106-00
6	.9630553-00	.9628477-00	.9621774-00	.9611461-00	.9519887-00
7	.9215870-00	.9214073-00	.9207746-00	.9197572-00	.9110012-00
8	.8649799-00	.8647932-00	.8642093-00	.8632811-00	.8550156-00
9	.7941176-00	.7939573-00	.7934054-00	.7925511-00	.7849783-00
10	.7100870-00	.7099364-00	.7094487-00	.7086613-00	.7018953-00
11	.6142483-00	.6141215-00	.6136992-00	.6130372-00	.6071506-00
12	.5049654-00	.5048498-00	.5045000-00	.5039495-00	.4991318-00
13	.3740843-00	.3739965-00	.3737378-00	.3733268-00	.3697648-00
14	.2557237-00	.2556457-00	.2554644-00	.2551989-00	.2527595-00
15	.1768979-00	.1767568-00	.1766346-00	.1764353-00	.1747561-00
16	.1212360+00	.1211996+00	.1211076+00	.1209706+00	.1198328+00
17	.8213459-01	.8210798-01	.8204861-01	.8197043-01	.8118136-01
18	.5386170-01	.5384623-01	.5381417-01	.5374742-01	.5323652-01
19	.3278026-01	.3277143-01	.3273916-01	.3270521-01	.3239885-01
20	.1508522-01	.1507836-01	.1507447-01	.1505970-01	.1491372-01
	6	7	8	9	10
1	.9632435-00	.9230350-00	.8689002-00	.8019929-00	.7238461-00
2	.9629202-00	.9227451-00	.8685908-00	.8017239-00	.7236064-00
3	.9622554-00	.9221473-00	.8680069-00	.8011665-00	.7231326-00
4	.9612813-00	.9212066-00	.8671595-00	.8003386-00	.7223925-00
5	.9522604-00	.9125327-00	.8589985-00	.7928382-00	.7155625-00
6	.9275754-00	.8888636-00	.8366854-00	.7722239-00	.6969577-00
7	.8876345-00	.8505630-00	.8006220-00	.7388776-00	.6667833-00
8	.8330732-00	.7982482-00	.7513225-00	.6933776-00	.6256313-00
9	.7647834-00	.7327875-00	.6896176-00	.6363356-00	.5741373-00
10	.6838216-00	.6551480-00	.6165115-00	.5687023-00	.5129564-00
11	.5915143-00	.5666732-00	.5331930-00	.4917610-00	.4433232-00
12	.4862562-00	.4658231-00	.4382421-00	.4041525-00	.3642139-00
13	.3602213-00	.3450672-00	.3246087-00	.2993286-00	.2696928-00
14	.2462481-00	.2358777-00	.2218843-00	.2045823-00	.1842821-00
15	.1702648-00	.1630063-00	.1534238-00	.1414311-00	.1274341-00
16	.1167406+00	.1118352+00	.1052000+00	.9699593-01	.8742753-01
17	.7909443-01	.7577150-01	.7128031-01	.6574823-01	.5931518-01

18	.5187190-01	.4969257-01	.4675886-01	.4315451-01	.3896345-01
19	.3156643-01	.3024271-01	.2846849-01	.2627719-01	.2374065-01
20	.1453031-01	.1392529-01	.1310426-01	.1210034-01	.1094963-01
	11	12	13	14	15
1	.6361837-00	.5410097-00	.4381390-00	.3189418-00	.2128196-00
2	.6359663-00	.5408872-00	.4379912-00	.3188355-00	.2127468-00
3	.6355343-00	.5405206-00	.4376958-00	.3186210-00	.2126040-00
4	.6349128-00	.5399576-00	.4372568-00	.3183022-00	.2123760-00
5	.6289021-00	.5348737-00	.4331125-00	.3152794-00	.2103767-00
6	.6124993-00	.5208972-00	.4218000-00	.3070444-00	.2048719-00
7	.5859691-00	.4982984-00	.4034588-00	.2936808-00	.1959591-00
8	.5496544-00	.4673644-00	.3784029-00	.2754323-00	.1837746-00
9	.5042881-00	.4285925-00	.3469087-00	.2524737-00	.1684333-00
10	.4503637-00	.3825471-00	.3094855-00	.2251717-00	.1502390-00
11	.3889489-00	.3297401-00	.2662880-00	.1935384-00	.1289354-00
12	.3193412-00	.2701874-00	.2165982-00	.1565285-00	.1050921+00
13	.2363606-00	.1997223-00	.1590731-00	.1177081+00	.8282429-01
14	.1614953-00	.1362102-00	.1091641+00	.8446820-01	.6325743+01
15	.1116493+00	.9442040-01	.7669421-01	.6034269-01	.4644917-01
16	.7670968-01	.6523789-01	.5349239-01	.4289293-01	.3373243-01
17	.5216205-01	.4452837-01	.3681526-01	.2977864-01	.2377770-01
18	.3431639-01	.2943309-01	.2450025-01	.2003979-01	.1611872+01
19	.2096513-01	.1802974-01	.1507018-01	.1237310-01	.1006868-01
20	.9669658-02	.8331944-02	.6993311-02	.5778430-02	.4685162-02
	16	17	18	19	20
1	.1439374-00	.9670494-01	.6426194-01	.4134772-01	.2468116-01
2	.1438886-00	.9666885-01	.6424759-01	.4132899-01	.2466865+01
3	.1437844-00	.9661129-01	.6420466-01	.4129239-01	.2466053-01
4	.1436436-00	.9652322-01	.6412411-01	.4125875-01	.2463485+01
5	.1422893-00	.9559075-01	.6352681-01	.4087116-01	.2439971-01
6	.1385571-00	.9309911-01	.6187023-01	.3980315-01	.2376388-01
7	.1325453-00	.8904872-01	.5918125-01	.3808221-01	.2274352-01
8	.1242794+00	.8351636-01	.5552525-01	.3574597-01	.2135013-01
9	.1139538+00	.7660265-01	.5096772-01	.3283479-01	.1962216-01
10	.1016233+00	.6841017-01	.4561415-01	.2942777-01	.1763128-01
11	.8747756-01	.5921972-01	.3963178-01	.2568666-01	.1542624-01
12	.7230852-01	.4941399-01	.3334601-01	.2175316-01	.1311704-01
13	.5795153-01	.4036715-01	.2746870-01	.1811952-01	.1096255-01
14	.4552272-01	.3238660-01	.2236776-01	.1485646-01	.9094264-02
15	.3458529-01	.2508672-01	.1772631-01	.1184803-01	.7329826-02
16	.2559708-01	.1886049-01	.1353697-01	.9223159-02	.5694604-02
17	.1846630-01	.1381287-01	.9966567-02	.6884469-02	.4301264-02
18	.1260488-01	.9615001-02	.7028025-02	.4853389-02	.3070191-02
19	.7957160-02	.6061430-02	.4486154-02	.3134061-02	.1976217-02
20	.3726299-02	.2892055-02	.2120094-02	.1484617-02	.9507971-03
	21				
1	.1115890-01				
2	.1116591-01				
3	.1115376-01				

4	.1114041-01
5	.1103550-01
6	.1075131-01
7	.1028664-01
8	.9659150-02
9	.8891360-02
10	.7987053-02
11	.7002327-02
12	.5978204-02
13	.5028931-02
14	.4152796-02
15	.3368065-02
16	.2668762-02
17	.1996130-02
18	.1426049-02
19	.9329756-03
20	.4505314-03

PERT-IV TEST CASE

(2DF FLUX INPUT, 2 GROUPS, 3 ZONES)

FISSION SOURCE\*ADJOINT FLUX

	1	2	3	4	5
1	.1000000+01	.9998566-00	.9991575-00	.9981228-00	.9887265-00
2	.9997082-00	.9995758-00	.9989677-00	.9978643-00	.9883608-00
3	.9991967-00	.9993066-00	.9982992-00	.9972069-00	.9876982-00
4	.9981490-00	.9979892-00	.9972818-00	.9962031-00	.9867193-00
5	.9887744-00	.9886256-00	.9879953-00	.9869360-00	.9775462-00
6	.9634279-00	.9632681-00	.9626160-00	.9615955-00	.9524663-00
7	.9224203-00	.9222937-00	.9216832-00	.9206657-00	.9119304-00
8	.8666416-00	.8664959-00	.8659325-00	.8650147-00	.8567460-00
9	.7971603-00	.7970429-00	.7965046-00	.7956617-00	.7880757-00
10	.7155165-00	.7154016-00	.7149272-00	.7141347-00	.7073398-00
11	.6236798-00	.6235873-00	.6231714-00	.6225864-00	.6165375-00
12	.5217756-00	.5216807-00	.5213290-00	.5207644-00	.5157970-00
13	.3996932-00	.3996127-00	.3993419-00	.3989080-00	.3951070-00
14	.2839814-00	.2839013-00	.2836999-00	.2834050-00	.2807031-00
15	.2014523-00	.2013986-00	.2012590-00	.2010318-00	.1991213-00
16	.1408999-00	.1408600-00	.1407549-00	.1405964-00	.1392727-00
17	.9684837-01	.9681921-01	.9674938-01	.9665492-01	.9572610-01
18	.6431098-01	.6429367-01	.6425497-01	.6417743-01	.6356540-01
19	.3966207-01	.3965219-01	.3961450-01	.3957171-01	.3920098+01
20	.1882821-01	.1882077-01	.1881450-01	.1879684-01	.1861417-01
	6	7	8	9	10
1	.9635367-00	.9236741-00	.8701791-00	.8042797-00	.7276684-00
2	.9632151-00	.9233965-00	.8698687-00	.8040108-00	.7274299-00
3	.9625498-00	.9228139-00	.8692865-00	.8034510-00	.7269614-00
4	.9615794-00	.9218796-00	.8684544-00	.8026194-00	.7262159-00
5	.9526340-00	.9132619-00	.8603525-00	.7951750-00	.7194023-00
6	.9281810-00	.8897997-00	.8382088-00	.7746911-00	.7008756-00
7	.8886799-00	.8519054-00	.8025043-00	.7416187-00	.6708557-00
8	.8348914-00	.8003097-00	.7538403-00	.6966636-00	.6300840-00
9	.7679667-00	.7360738-00	.6932179-00	.6405302-00	.5793174+00
10	.6892152-00	.6605636-00	.6220560-00	.5745450-00	.5194518-00
11	.6007419-00	.5757158-00	.5420802-00	.5005631-00	.4522364+00
12	.5025571-00	.4816016-00	.4533828-00	.4186036-00	.3780020-00
13	.3849568-00	.3688740-00	.3472105-00	.3205036-00	.2893081-00
14	.2735900-00	.2620483-00	.2466231-00	.2275867-00	.2053196-00
15	.1940197-00	.1858806-00	.1749370-00	.1613799-00	.1455845-00
16	.1356880-00	.1300092-00	.1223389+00	.1128624+00	.1018320+00
17	.9326935-01	.8936444-01	.8409075-01	.7760174-01	.7007153-01

18	.6193794-01	.5934339-01	.5585258-01	.5157118-01	.4659590-01
19	.3819477-01	.3659673-01	.3445790-01	.3181697-01	.2876049-01
20	.1813611-01	.1738289-01	.1636055-01	.1511202-01	.1368294-01
	11	12	13	14	15
1	.6423965-00	.5509008-00	.4542062-00	.3419633-00	.2371703-00
2	.6421776-00	.5507179-00	.4540512-00	.3418500-00	.2370899-00
3	.6417480-00	.5503446-00	.4537468-00	.3416208-00	.2369303-00
4	.6411326-00	.5497735-00	.4532957-00	.3412811-00	.2366796-00
5	.6350998-00	.5446406-00	.4490314-00	.3380629-00	.2344624-00
6	.6186818-00	.5305244-00	.4373947-00	.3292923-00	.2283628-00
7	.5921705-00	.5077493-00	.4185585-00	.3150896-00	.2184994-00
8	.5559800-00	.4766391-00	.3928962-00	.2957295-00	.2050385-00
9	.5110376-00	.4378435-00	.3607635-00	.2714719-00	.1881486-00
10	.4580285-00	.3921477-00	.3228864-00	.2428047-00	.1682050-00
11	.3984012-00	.3402864-00	.2793162-00	.2098494-00	.1449993-00
12	.3327061-00	.2834236-00	.2307304-00	.1717388-00	.1190996+00
13	.2544077-00	.2163040-00	.1744356-00	.1317280-00	.9479888-01
14	.1804157-00	.1529257-00	.1235730+00	.9662477-01	.7325826-01
15	.1278303-00	.1084975+00	.8860752-01	.7024148-01	.5450085-01
16	.8950693-01	.7633114-01	.6283389-01	.5060665-01	.4000641-01
17	.6170773-01	.5277882-01	.4374758-01	.3551077-01	.2845693-01
18	.4107843-01	.3528243-01	.2942965-01	.2412146-01	.1945406-01
19	.2542017-01	.2188833-01	.1832217-01	.1507516-01	.1228887-01
20	.1209317-01	.1042903-01	.8764614-02	.7250180-02	.5894209-02
	16	17	18	19	20
1	.1645175-00	.1127196+00	.7596333-01	.4947530-01	.2992415-01
2	.1644616-00	.1126786+00	.7594550-01	.4945305-01	.2990956-01
3	.1643443-00	.1126113+00	.7589521-01	.4940987-01	.2989837-01
4	.1641341-00	.1125071+00	.7580222-01	.4936819-01	.2986840-01
5	.1626413-00	.1114259+00	.7509625-01	.4890640-01	.2958365-01
6	.1583963-00	.1085321+00	.7314515-01	.4763207-01	.2881449-01
7	.1515628-00	.1038346+00	.6997808-01	.4557920-01	.2758152-01
8	.1421856-00	.9742277-01	.6567759-01	.4279743-01	.2589831-01
9	.1304953-00	.8942962-01	.6032924-01	.3933367-01	.2381155-01
10	.1165911+00	.7998497-01	.5405531-01	.3528023-01	.2141107-01
11	.1006870+00	.6940648-01	.4704387-01	.3083228-01	.1875368-01
12	.8364575-01	.5812072-01	.3967228-01	.2615945-01	.1596663-01
13	.6751579-01	.4707075-01	.3278747-01	.2182906-01	.1337088-01
14	.5343499-01	.3843198-01	.2678723-01	.1794243-01	.1110899-01
15	.4091229-01	.2993135-01	.2130376-01	.1435142-01	.8970361-02
16	.3053263-01	.2262247-01	.1633442-01	.1120330-01	.6989556-02
17	.2218456-01	.1666458-01	.1207609-01	.8391491-02	.5292970-02
18	.1526119-01	.1167612-01	.8565616-02	.5946467-02	.3795627-02
19	.9731747-02	.7435973-02	.5518396-02	.3873665-02	.2465255-02
20	.4697828-02	.3659536-02	.2687149-02	.1888795-02	.1218774-02
	21				
1	.1395338-01				
2	.1396063-01				
3	.1394663-01				

4	.1392956-01
5	.1379907-01
6	.1344455-01
7	.1286480-01
8	.1208266-01
9	.1112755-01
10	.1000244-01
11	.8775192-02
12	.7500386-02
13	.6314985-02
14	.5228608-02
15	.4248862-02
16	.3369931-02
17	.2530729-02
18	.1814705-02
19	.1195761-02
20	.5949106-03



PERT-IV TEST CASE

(2DF FLUX INPUT, 2 GROUPS, 3 ZONES)

POWER DENSITY (MWT/LITER)

	1	2	3	4	5
1	.1232984-15	.1232733-15	.1231851-15	.1230549-15	.1218897-15
2	.1232589-15	.1232351-15	.1231565-15	.1230204-15	.1218457-15
3	.1231899-15	.1231608-15	.1230720-15	.1229376-15	.1217639-15
4	.1230617-15	.1230356-15	.1229465-15	.1228125-15	.1216419-15
5	.1218992-15	.1218743-15	.1217930-15	.1216606-15	.1205005-15
6	.1187427-15	.1187170-15	.1186343-15	.1185071-15	.1173781-15
7	.1136292-15	.1136069-15	.1135288-15	.1134034-15	.1123238-15
8	.1066486-15	.1066254-15	.1065534-15	.1064389-15	.1054199-15
9	.9790969-16	.9788979-16	.9782170-16	.9771635-16	.9678274-16
10	.8754587-16	.8752719-16	.8746701-16	.8736993-16	.8653581-16
11	.7572469-16	.7570896-16	.7565686-16	.7557525-16	.7484960-16
12	.6224442-16	.6223008-16	.6218693-16	.6211907-16	.6152526-16
13	.1420256-16	.1419976-16	.1419013-16	.1417456-16	.1403901-16
14	.9771452-17	.9768704-17	.9761882-17	.9751752-17	.9658325-17
15	.6799078-17	.6797260-17	.6792591-17	.6784838-17	.6720208-17
16	.4686312-17	.4684999-17	.4681436-17	.4676144-17	.4632109-17
17	.3187558-17	.3186571-17	.3184292-17	.3181281-17	.3150555-17
18	.2100514-17	.2099966-17	.2098737-17	.2096107-17	.2076116-17
19	.1288134-17	.1287801-17	.1286506-17	.1285155-17	.1273129-17
20	.6076100-18	.6073578-18	.6072192-18	.6066417-18	.6007301-18
	6	7	8	9	10
1	.1187662-15	.1138082-15	.1071327-15	.9888191-16	.8924468-16
2	.1187263-15	.1137724-15	.1070946-15	.9884875-16	.8921513-16
3	.1186443-15	.1136987-15	.1070226-15	.9878003-16	.8915670-16
4	.1185242-15	.1135827-15	.1069181-15	.9867796-16	.8906547-16
5	.1174119-15	.1125132-15	.1059118-15	.9775310-16	.8822330-16
6	.1143680-15	.1095945-15	.1031604-15	.9521122-16	.8592927-16
7	.1094420-15	.1048716-15	.9871346-16	.9109932-16	.8220861-16
8	.1027145-15	.9842040-16	.9263401-16	.8548854-16	.7713414-16
9	.9429286-16	.9034769-16	.8502456-16	.7845425-16	.7078410-16
10	.8430753-16	.8077217-16	.7600823-16	.7011308-16	.6323895-16
11	.7292195-16	.6985934-16	.6573149-16	.6062308-16	.5465064-16
12	.5993816-16	.5741933-16	.5401924-16	.4981667-16	.4489284-16
13	.1367664-16	.1310224-16	.1232769-16	.1137160-16	.1025194-16
14	.9409541-17	.9013848-17	.8480549-17	.7821566-17	.7049236-17
15	.6547517-17	.6271844-17	.5901166-17	.5441209-17	.4904905-17
16	.4512524-17	.4323204-17	.4067202-17	.3750757-17	.3382078-17
17	.3069548-17	.2940757-17	.2766679-17	.2552480-17	.2303541-17

18	.2022911-17	.1937979-17	.1823741-17	.1683522-17	.1520374-17
19	.1240395-17	.1188418-17	.1118840-17	.1032836-17	.9332967-18
20	.5852792-18	.5609372-18	.5278842-18	.4875109-18	.4412514-18
	11	12	13	14	15
1	.7843313-16	.6670167-16	.5400543-16	.1212234-16	.8141006-17
2	.7840632-16	.6667916-16	.5398722-16	.1211831-16	.8138223-17
3	.7835305-16	.6663397-16	.5395080-16	.1211018-16	.8132738-17
4	.7827641-16	.6656456-16	.5389669-16	.1209804-16	.8124006-17
5	.7753533-16	.6593778-16	.5338581-16	.1198345-16	.8047754-17
6	.7551290-16	.6421465-16	.5199132-16	.1167123-16	.7837595-17
7	.7224175-16	.6142844-16	.4973034-16	.1116479-16	.7497635-17
8	.6776403-16	.5761451-16	.4664155-16	.1047381-16	.7033013-17
9	.6216992-16	.5283399-16	.4275891-16	.9605414-17	.6448714-17
10	.5551997-16	.4715620-16	.3814497-16	.8575225-17	.5756678-17
11	.4794573-16	.4064424-16	.3281895-16	.7381502-17	.4946016-17
12	.3936034-16	.3329980-16	.2669195-16	.5988554-17	.4039860-17
13	.8995221-17	.7614191-17	.6084869-17	.4524165-17	.3194231-17
14	.6183332-17	.5222242-17	.4194896-17	.3256914-17	.2445907-17
15	.4300383-17	.3640271-17	.2961184-17	.2334306-17	.1799932-17
16	.2969120-17	.2526996-17	.2074372-17	.1665773-17	.1311724-17
17	.2026597-17	.1730925-17	.1432085-17	.1159378-17	.9269056-18
18	.1339401-17	.1149268-17	.9572611-18	.7835343-18	.6305037-18
19	.8244304-18	.7092929-18	.5931295-18	.4872755-18	.3968928-18
20	.3897777-18	.3359361-18	.2820708-18	.2331581-18	.1891412-18
	16	17	18	19	20
1	.5540133-17	.3740779-17	.2495257-17	.1613177-17	.9702839-18
2	.5538244-17	.3739395-17	.2494741-17	.1612393-17	.9697918-18
3	.5534236-17	.3737226-17	.2493056-17	.1610935-17	.9694954-18
4	.5528901-17	.3733823-17	.2489881-17	.1609638-17	.9685137-18
5	.5476824-17	.3697740-17	.2466751-17	.1594584-17	.9592488-18
6	.5333433-17	.3601572-17	.2402561-17	.1552948-17	.9342771-18
7	.5102651-17	.3445197-17	.2298266-17	.1485920-17	.8942561-18
8	.4785300-17	.3231637-17	.2156652-17	.1395009-17	.8395413-18
9	.4389347-17	.2965082-17	.1980202-17	.1281623-17	.7716997-18
10	.3916703-17	.2649197-17	.1772789-17	.1148890-17	.6935834-18
11	.3374265-17	.2294749-17	.1540983-17	.1003197-17	.6070648-18
12	.2792875-17	.1916747-17	.1297390-17	.8500732-18	.5164021-18
13	.2242392-17	.1567985-17	.1069596-17	.7085421-18	.4318410-18
14	.1764329-17	.1259579-17	.8720217-18	.5811742-18	.3585791-18
15	.1342007-17	.9761131-18	.6920364-18	.4637994-18	.2891591-18
16	.9958270-18	.7342181-18	.5286848-18	.3616403-18	.2246880-18
17	.7208500-18	.5393811-18	.3894813-18	.2702633-18	.1699957-18
18	.4934121-18	.3769844-18	.2758675-18	.1909925-18	.1217535-18
19	.3138265-18	.2391171-18	.1772748-18	.1242596-18	.7888532-19
20	.1506328-18	.1170665-18	.8584845-19	.6024071-19	.3886715-19
	21				
1	.4495929-18				
2	.449207-18				
3	.4494437-18				

4	.4488739-18
5	.4446553-18
6	.4332292-18
7	.4145158-18
8	.3892744-18
9	.3584034-18
10	.3220258-18
11	.2823815-18
12	.2411692-18
13	.2029402-18
14	.1676656-18
15	.1361704-18
16	.1080491-18
17	.8084084-19
18	.5786321-19
19	.3813085-19
20	.1890341-19

PERT-IV TEST CASE

(2DF FLUX INPUT, 2 GROUPS, 3 ZONES)

ACTIVITY 1	ISOTOPE 1	CROSS SECTION POSITION 1				
	1	2	3	4	5	
1	.1907972+01	.1907564+01	.1906193+01	.1904174+01	.1886147+01	
2	.1907354+01	.1906968+01	.1905745+01	.1903638+01	.1885468+01	
3	.1906278+01	.1905812+01	.1904432+01	.1902354+01	.1884203+01	
4	.1904295+01	.1903875+01	.1902490+01	.1900416+01	.1882314+01	
5	.1886302+01	.1885899+01	.1884633+01	.1882583+01	.1864640+01	
6	.1837425+01	.1837910+01	.1835725+01	.1833756+01	.1816291+01	
7	.1758240+01	.1757877+01	.1756663+01	.1754722+01	.1738025+01	
8	.1650116+01	.1649742+01	.1648621+01	.1646849+01	.1631091+01	
9	.1514715+01	.1514393+01	.1513335+01	.1511703+01	.1497267+01	
10	.1354039+01	.1353738+01	.1352803+01	.1351301+01	.1338405+01	
11	.1170667+01	.1170414+01	.1169604+01	.1168342+01	.1157129+01	
12	.9614747-00	.9612448-00	.9605753-00	.9595262-00	.9503582-00	
13	.7113178-00	.7111445-00	.7106505-00	.7098685-00	.7030990-00	
14	.4855099-00	.4853590-00	.4850135-00	.4845093-00	.4798805-00	
15	.3351690-00	.3350733-00	.3348383-00	.3344615-00	.3312791-00	
16	.2295342-00	.2294642-00	.2292902-00	.2290308-00	.2268771-00	
17	.1553528-00	.1553020-00	.1551894-00	.1550412-00	.1535500-00	
18	.1017547+00	.1017248+00	.1016640+00	.1015382+00	.1005738+00	
19	.6181158-01	.6179478-01	.6173424-01	.6167043-01	.6109259-01	
20	.2826849-01	.2825536-01	.2824785-01	.2821996-01	.2794678-01	
	6	7	8	9	10	
1	.1837811+01	.1761057+01	.1657682+01	.1529882+01	.1380564+01	
2	.1837195+01	.1760502+01	.1657093+01	.1529369+01	.1380106+01	
3	.1835927+01	.1759359+01	.1655978+01	.1528306+01	.1379202+01	
4	.1834068+01	.1757563+01	.1654359+01	.1526727+01	.1377791+01	
5	.1816846+01	.1741906+01	.1638779+01	.1512408+01	.1364756+01	
6	.1769713+01	.1695816+01	.1596181+01	.1473057+01	.1329247+01	
7	.1693443+01	.1622680+01	.1527321+01	.1409392+01	.1271653+01	
8	.1589230+01	.1522760+01	.1433165+01	.1322498+01	.1193078+01	
9	.1458748+01	.1397687+01	.1315280+01	.1213534+01	.1094723+01	
10	.1303943+01	.1249239+01	.1175509+01	.1084250+01	.9777997-00	
11	.1127329+01	.1079964+01	.1016107+01	.9370678-00	.8446331-00	
12	.9258428-00	.8869208-00	.8343673-00	.7693975-00	.6932588-00	
13	.6849527-00	.6561260-00	.6171983-00	.5690844-00	.5126668-00	
14	.4675180-00	.4478222-00	.4212380-00	.3883634-00	.3497820-00	
15	.3227648-00	.3091523-00	.2908249-00	.2680761-00	.2415192-00	
16	.2210234-00	.2117327-00	.1991645-00	.1836238-00	.1654940-00	
17	.1496028-00	.1433157-00	.1348182-00	.1243487+00	.1121724+00	

18	.9799564-01	.9387780-01	.8633344-01	.8152003-01	.7359881-01
19	.5952324-01	.5702670-01	.5367948-01	.4954623-01	.4476161-01
20	.2722840-01	.2609431-01	.2455555-01	.2267355-01	.2051615-01
	11	12	13	14	15
1	.1212966+01	.1031518+01	.8340428-00	.6063061-00	.4039465-00
2	.1212551+01	.1030670+01	.8337615-00	.6061038-00	.4038084-00
3	.1211726+01	.1029972+01	.8331990-00	.6056958-00	.4035375-00
4	.1210540+01	.1028898+01	.8323633-00	.6050899-00	.4031049-00
5	.1199075+01	.1019205+01	.8244689-00	.5993401-00	.3993074-00
6	.1167780+01	.9925559-00	.8029216-00	.5836763-00	.3888540-00
7	.1117158+01	.9494602-00	.7679814-00	.5582545-00	.3719251-00
8	.1047851+01	.8904602-00	.7202407-00	.5235330-00	.3487805-00
9	.9612321-00	.8164840-00	.6602146-00	.4798385-00	.3196312-00
10	.8582114-00	.7285764-00	.5888456-00	.4278482-00	.2850500-00
11	.7408136-00	.6277133-00	.5064451-00	.3676110-00	.2445635-00
12	.6076701-00	.5138909-00	.4116000-00	.2970919-00	.1992366-00
13	.4491811-00	.3793945-00	.3019339-00	.2231622-00	.1568970-00
14	.3064623-00	.2583961-00	.2069742-00	.1600196-00	.1197556+00
15	.2115671-00	.1788779-00	.1452444-00	.1142245+00	.8788827-01
16	.1451862-00	.1234512+00	.1011970+00	.8111586-01	.6377189-01
17	.9863476-01	.8418915-01	.6959432-01	.5628046-01	.4492506-01
18	.6481664-01	.5558752-01	.4626411-01	.3783481-01	.3042857-01
19	.3952561-01	.3398801-01	.2840575-01	.2331848-01	.1897113-01
20	.1811661-01	.1560937-01	.1310024-01	.1082340-01	.8774492-02
	16	17	18	19	20
1	.2727968-00	.1830578-00	.1215320+00	.7810529-01	.4653472-01
2	.2727045-00	.1829893-00	.1215044+00	.7807055-01	.4651116-01
3	.2725069-00	.1828796-00	.1214234+00	.7800178-01	.4649556-01
4	.2722392-00	.1827129-00	.1212717+00	.7793804-01	.4644680-01
5	.2696718-00	.1809479-00	.1201413+00	.7720508-01	.4600374-01
6	.2625952-00	.1762288-00	.1170068+00	.7518723-01	.4480461-01
7	.2511941-00	.1685580-00	.1119200+00	.7193503-01	.4287969-01
8	.2355186-00	.1580802-00	.1050016+00	.6751909-01	.4025179-01
9	.2159315-00	.1449823-00	.9637629-01	.6201759-01	.3699276-01
10	.1925388-00	.1294622-00	.8624610-01	.5557949-01	.3323727-01
11	.1657050-00	.1120525+00	.7492647-01	.4850942-01	.2907781-01
12	.1369265-00	.9347513-01	.6303309-01	.4107503-01	.2472255-01
13	.1096912+00	.7633570-01	.5191296-01	.3420831-01	.2065877-01
14	.8613173-01	.6122533-01	.4226020-01	.2804516-01	.1713402-01
15	.6541860-01	.4742000-01	.3347944-01	.2236228-01	.1380795-01
16	.4838641-01	.3564654-01	.2556471-01	.1740097-01	.1072706-01
17	.3487796-01	.2608667-01	.1881912-01	.1298480-01	.8099008-02
18	.2379097-01	.1614048-01	.1325594-01	.9148456-02	.5776055-02
19	.1499268-01	.1141859-01	.8447481-02	.5896537-02	.3711782-02
20	.6976309-02	.5412576-02	.3967471-02	.2776777-02	.1774915-02
	21				
1	.2090934-01				
2	.2092194-01				
3	.2089901-01				

4	.2087438-01
5	.2067770-01
6	.2014491-01
7	.1927412-01
8	.1809787-01
9	.1665846-01
10	.1496329-01
11	.1311777-01
12	.1119819-01
13	.9419254-02
14	.7777269-02
15	.6305405-02
16	.4994424-02
17	.3735344-02
18	.2667245-02
19	.1741740-02
20	.8352372-03

PERT-IV TEST CASE

(2DF FLUX INPUT, 2 GROUPS, 3 ZONES)

ZONE	AV FLUX	AV POWER	ACT 1
1	.9980257-00	.1230547-15	.1904171+01
2	.5448032-00	.6716426-16	.1038406+01
3	.5995134-01	.2308208-17	.1136144+00

PERT-IV TEST CASE

(2DF FLUX INPUT, 2 GROUPS, 3 ZONES)

BETA EFFECTIVE - - - - - 4.20647-03

NEUTRON GENERATION TIME - - - - - 3.02887-07



PERT-IV TEST CASE

(2DF FLUX INPUT, 2 GROUPS, 3 ZONES)

REACTIVITY COEFFICIENTS FOR MATERIAL 1 M<sub>2</sub>PU49

ROW 1

	ΔK/K PER KG	ΔK (NUSIGF)	ΔK (SIGA)	ΔK (SIGTR)	ΔK (SIGJXK)	ΔK (INTG)	AVG RADII	AVG AXII
1	.5455-02	.8937-02	-.3469-02	.2461-07	-.1278-04	.5455-02	.3175-00	.3175-00
2	.5454-02	.8933-02	-.3467-02	.7413-07	-.1279-04	.5454-02	.9525-00	.3175-00
3	.5446-02	.8921-02	-.3462-02	.2113-06	-.1277-04	.5450-02	.1587+01	.3175-00
4	.5435-02	.8902-02	-.3455-02	.7864-06	-.1275-04	.5443-02	.2222+01	.3175-00
5	.5335-02	.8735-02	-.3389-02	.2424-05	-.1253-04	.5350-02	.4716+01	.3175-00
6	.5071-02	.8294-02	-.3217-02	.5893-05	-.1197-04	.5176-02	.9068+01	.3175-00
7	.4667-02	.7619-02	-.2953-02	.1216-04	-.1113-04	.4932-02	.1342+02	.3175-00
8	.4152-02	.6756-02	-.2614-02	.1998-04	-.1009-04	.4629-02	.1777+02	.3175-00
9	.3560-02	.5763-02	-.2223-02	.2866-04	-.8949-05	.4280-02	.2212+02	.3175-00
10	.2930-02	.4706-02	-.1806-02	.3741-04	-.7795-05	.3901-02	.2648+02	.3175-00
11	.2301-02	.3650-02	-.1387-02	.4538-04	-.6744-05	.3507-02	.3083+02	.3175-00
12	.1714-02	.2661-02	-.9937-03	.5312-04	-.5844-05	.3114-02	.3518+02	.3175-00
13	.1203-02	.1775-02	-.6390-03	.7166-04	-.5188-05	.2736-02	.3953+02	.3175-00
14	.6701-03	.9717-03	-.3263-03	.2894-04	-.4143-05	.2404-02	.4361+02	.3175-00
15	.3264-03	.4491-03	-.1389-03	.1881-04	-.2610-05	.2096-02	.4742+02	.3175-00
16	.1557-03	.2104-03	-.6156-04	.8306-05	-.1445-05	.1828-02	.5123+02	.3175-00
17	.7281-04	.9677-04	-.2713-04	.3926-05	-.7430-06	.1601-02	.5504+02	.3175-00
18	.3304-04	.4330-04	-.1180-04	.1900-05	-.3566-06	.1410-02	.5885+02	.3175-00
19	.1410-04	.1813-04	-.4846-05	.9712-06	-.1572-06	.1250-02	.6266+02	.3175-00
20	.5307-05	.6534-05	-.1732-05	.5630-06	-.5904-07	.1115-02	.6647+02	.3175-00
21	.1440-05	.1370-05	-.3675-06	.4501-06	-.1316-07	.1001-02	.7028+02	.3175-00

PERT-IV TEST CASE

(2DF FLUX INPUT, 2 GROUPS, 3 ZONES)

REACTIVITY COEFFICIENTS FOR MATERIAL 1 M2PU49 COLUMN 5

	ΔK/K PER KG	ΔK (NUSIGF)	ΔK (SIGA)	ΔK (SIGTR)	ΔK (SIGJXK)	ΔK (INTG)	AVG RADII	AVG AXII
1	.5335-02	.8735-02	-.3389-02	.2424-05	+.1253-04	.5335-02	.4716+01	.3175-00
2	.5331-02	.8728-02	-.3387-02	.2472-05	-.1252-04	.5333-02	.4716+01	.9525-00
3	.5324-02	.8717-02	-.3382-02	.2579-05	-.1251-04	.5330-02	.4716+01	.1587+01
4	.5314-02	.8699-02	-.3376-02	.3094-05	-.1249-04	.5326-02	.4716+01	.2222+01
5	.5218-02	.8538-02	-.3312-02	.4597-05	-.1229-04	.5257-02	.4716+01	.4767+01
6	.4957-02	.8103-02	-.3142-02	.7954-05	-.1175-04	.5140-02	.4716+01	.9222+01
7	.4552-02	.7424-02	-.2875-02	.1428-04	-.1093-04	.4976-02	.4716+01	.1368+02
8	.4028-02	.6545-02	-.2530-02	.2246-04	-.9924-05	.4768-02	.4716+01	.1813+02
9	.3423-02	.5527-02	-.2127-02	.3197-04	-.8838-05	.4527-02	.4716+01	.2258+02
10	.2776-02	.4435-02	-.1693-02	.4220-04	-.7817-05	.4260-02	.4716+01	.2704+02
11	.2134-02	.3342-02	-.1255-02	.5403-04	-.6953-05	.3979-02	.4716+01	.3149+02
12	.1538-02	.2297-02	-.8325-03	.7973-04	-.6400-05	.3695-02	.4716+01	.3595+02
13	.8904-03	.1302-02	-.4404-03	.3430-04	-.5366-05	.3440-02	.4716+01	.4008+02
14	.4544-03	.6314-03	-.1968-03	.2339-04	-.3575-05	.3192-02	.4716+01	.4389+02
15	.2270-03	.3093-03	-.9115-04	.1090-04	-.2081-05	.2964-02	.4716+01	.4770+02
16	.1107-03	.1482-03	-.4183-04	.5422-05	-.1119-05	.2760-02	.4716+01	.5151+02
17	.5224-04	.6894-04	-.1889-04	.2753-05	-.5605-06	.2580-02	.4716+01	.5532+02
18	.2315-04	.2999-04	-.8056-05	.1476-05	-.2575-06	.2421-02	.4716+01	.5913+02
19	.9054-05	.1124-04	-.2990-05	.9068-06	-.1007-06	.2279-02	.4716+01	.6294+02
20	.2514-05	.2443-05	-.6573-06	.7512-06	-.2331-07	.2153-02	.4716+01	.6675+02

PERT-IV TEST CASE

(2DF FLUX INPUT, 2 GROUPS, 3 ZONES)

REACTIVITY COEFFICIENTS FOR MATERIAL 6 NA ROW 1

	ΔK/K PER KG	ΔK (MUSIGF)	ΔK (SIGA)	ΔK (SIGTR)	ΔK (SIGJXK)	ΔK (INTG)	AVG RADII	AVG AXII
1	-.1773-03	.0000	-.2791-04	.1112-06	-.1495-03	-.1773-03	.3175-00	.3175-00
2	-.1772-03	.0000	-.2789-04	.3334-06	-.1496-03	-.1772-03	.9525-00	.3175-00
3	-.1764-03	.0000	-.2785-04	.9636-06	-.1495-03	-.1767-03	.1587+01	.3175-00
4	-.1734-03	.0000	-.2779-04	.3601-03	-.1492-03	-.1753-03	.2222+01	.3175-00
5	-.1628-03	.0000	-.2726-04	.1111-04	-.1467-03	-.1645-03	.4716+01	.3175-00
6	-.1389-03	.0000	-.2587-04	.2699-04	-.1400-03	-.1485-03	.9068+01	.3175-00
7	-.9827-04	.0000	-.2373-04	.5565-04	+.1302-03	-.1244-03	.1342+02	.3175-00
8	-.4767-04	.0000	-.2098-04	.9142-04	-.1181-03	-.9457-04	.1777+02	.3175-00
9	.8471-05	.0000	-.1779-04	.1310-03	-.1047-03	-.6097-04	.2212+02	.3175-00
10	.6510-04	.0000	-.1439-04	.1707-03	-.9122-04	-.2558-04	.2648+02	.3175-00
11	.1168-03	.0000	-.1096-04	.2066-03	-.7892-04	.9486-05	.3083+02	.3175-00
12	.1650-03	.0000	-.7729-05	.2411-03	-.6839-04	.4361-04	.3518+02	.3175-00
13	.2593-33	.0000	-.4804-05	.3248-03	-.6071-04	.8628-04	.3953+02	.3175-00
14	.8034-04	.0000	-.2291-05	.1311-03	-.4849-04	.8532-04	.4361+02	.3175-00
15	.5468-04	.0000	-.8857-06	.8611-04	-.3055-04	.8077-04	.4742+02	.3175-00
16	.2123-04	.0000	-.3626-06	.3850-04	-.1691-04	.7254-04	.5123+02	.3175-00
17	.9517-05	.0000	-.1490-06	.1836-04	-.8695-05	.6439-04	.5504+02	.3175-00
18	.4713-05	.0000	-.6145-07	.8948-03	-.4174-05	.5714-04	.5885+02	.3175-00
19	.2731-05	.0000	-.2416-07	.4593-05	-.1840-05	.5091-04	.6266+02	.3175-00
20	.1974-05	.0000	-.8338-08	.2673-05	-.6909-06	.4561-04	.6647+02	.3175-00
21	.1984-05	.0000	-.1697-08	.2140-03	-.1540-06	.4112-04	.7028+02	.3175-00

APPENDIX III: LISTING OF PERT-IV

```

-IL PDP INCL
ABC* FCOPY
  REAL I2
  DIMENSION  A(25,10), A0(50), A1(50), A2(50), ADJ(50,50), ATW(25)
1  , BETA(20), C(15,26,35), CO(15,35), CHI(26), CHIP(26), DUM(2500),
2  HOLN(25), PHC(26), PHI(50,50), PHR(50,50), PHZ(50,50), R1(51),
3  R4(50), R5(50), SLOPR(50), SLOPZ(50), SORC1(50,50), SORC2(50,50),
4  V0(50,50), VEL(26), VOL(40), Z1(51), Z4(50), Z5(50), ZACT(40,3),
5  ZPHI(40), ZPOW(40)
  DIMENSION  I0(45), I1(45), I2(45), ID(12), L2(40), LDUM(2500),
1  M0(50,50), M2(40), MA(15), MATDK(20), NBET(20), NCOL(20),
2  NROW(20), NX(15)
  COMMON  NINP, NOUT, NCR1, NFLUX1, NSCRAT, NFLUX2,
1  IFP, IGM, IGE, IGEP, IM, IMJM, IP, IST, ITL, IZM, JM, JP, M01,
2  MCR, MT, MTP, NACT, ND, NDELK, NFP, NIBC, NPRT
  COMMON  AK, DENOM, FEF, FLPO, VF, ZKEFF
  COMMON  A, A0, A1, A2, ADJ, ATW, BETA, C, CO, CHI, CHIP, DUM,
1  HOLN, PHC, PHI, PHR, PHZ, R1, R4, R5, SLOPR, SLOPZ, SORC1, SORC2,
2  V0, VEL, VOL, Z1, Z4, Z5, ZACT, ZPHI, ZPOW
  COMMON  I0, I1, I2, ID, L2, LDUM, M0, M2, MA, MATDK, NBET, NCOL,
1  NROW, NX
  END

```

-ITC FOR MAIN,MAIN  
INCLUDE ABC

C  
C MAIN MAIN PROGRAM SETS UP TAPE UNITS AND CALLS INP1, MAPR,  
C SETUP, GRAM, NORM, ABETA, AND CALC.  
C  
C INP1 SUBROUTINE TO CONTROL THE READING AND PRINTING OF ALL  
C INPUT DATA. INP1 IS CALLED BY MAIN AND CALLS XSINP,  
C FXINP, REAG2, AND REAI2.  
C  
C XSINP XSINP READS, CHECKS, PRINTS, AND WRITES THE CROSS  
C SECTIONS TO TAPE. THE SUBROUTINE IS CALLED BY INP1.  
C  
C FXINP SUBROUTINE TO READ AND PRINT THE INPUT FLUXES. IF FLUXES  
C ARE IN 1-D FORM, THEY ARE COMBINED RESULTING IN THE 2-D  
C FORM. INP1 CALLS FXINP, REAG2, AND REAI2 AND IS CALLED  
C BY MAIN.  
C  
C REAI2 REAI2 READS ALL INTEGER DATA AFTER CARD 4. IT IS CALLED  
C BY FXINP AND INP1.  
C  
C REAG2 REAG2 READS ALL REAL DATA AFTER THE CROSS SECTIONS EXCEPT  
C FOR THE 2-D FLUXES. IT IS CALLED BY FXINP AND INP1.  
C  
C MAPR SUBROUTINE TO PRODUCE A PICTURE PRINT BY ZONE AND BY  
C MATERIAL. MAPR IS CALLED BY MAIN.  
C  
C PRT SUBROUTINE PRINTS OUT TWO-DIMENSIONAL ARRAYS. IT IS  
C CALLED BY NORM.  
C  
C ERRO2 SUBROUTINE TO PRINT AN ERROR MESSAGE. ERRO2 IS CALLED  
C BY REAI2 AND REAG2.  
C  
C SETUP SETUP MIXES THE CROSS SECTIONS AND CALCULATES THE AREAS  
C AND VOLUMES. IT IS CALLED BY MAIN.  
C  
C GRAM THIS SUBROUTINE CALCULATES THE MASS OF EACH MATERIAL IN  
C EACH ZONE AND ALSO THE ZONE VOLUME. IT IS CALLED BY  
C MAIN.  
C  
C NORM NORM NORMALIZES THE FLUX AND ADJOINT FLUX AND CALCULATES  
C K EFFECTIVE. IT CALLS PRT AND IS CALLED BY MAIN.  
C  
C AVG THIS SUBROUTINE CALCULATES ZONE AVERAGED FLUXES, POWERS,  
C AND ACTIVITIES. IT IS CALLED BY NORM.  
C  
C ABETA ABETA PERFORMS THE NEUTRON LIFETIME AND BETA EFFECTIVE

```

C          CALCULATION.  IT IS CALLED BY MAIN.
C
C  CALC      SUBROUTINE TO CALCULATE AND PRINT REACTIVITY
C             COEFFICIENTS.  IT IS CALLED BY MAIN.
C
C  AK        CALCULATED VALUE OF K EFFECTIVE
C  CAD       TOTAL CHI*ADJOINT AT CENTER
C  CFX       TOTAL CENTER POINT FLUX
C  CPW       TOTAL CENTER POINT POWER
C  DENOM     DENOMINATOR OF PERTURBATION FORMULA
C  FEF       AVERAGE ENERGY RELEASED PER FISSION (MEV/FISSION)
C  FLPO      FLUX OR POWER (SEE NFP)
C  IFP       FLUX PUNCH OPTION
C  IGE       TYPE OF GEOMETRY      (0/1/2=R-Z/X-Y/R-THETA)
C  IGEP      IGE + 1
C  IGM       NUMBER OF GROUPS      (< OR = 26)
C  IGP       NUMBER OF GROUPS PLUS 1
C  IM        NUMBER OF RADIAL INTERVALS      (< OR = 50)
C  IMJM      IM*JM
C  IP        IM + 1
C  IST       NUMBER OF DOWNSCATTERING TERMS  (< OR = 10)
C  ITL       CROSS SECTION TABLE LENGTH    (< OR = 15)
C  IZM       NUMBER OF MATERIAL ZONES      (< OR = 40)
C  JM        NUMBER OF AXIAL INTERVALS      (< OR = 50)
C  JP        JM + 1
C  M01       LENGTH OF MIX VECTORS (I0,I1,I2)      (< OR = 45)
C  MCR       NUMBER OF MATERIALS FROM CARDS  (< OR = 25)
C  MT        TOTAL NUMBER OF MATERIALS (< OR = 35)
C  MTP       NUMBER OF MATERIALS FROM TAPE
C  NACT      NUMBER OF ACTIVITY TRAVERSES  (< OR = 3)
C  NCR1      CROSS SECTION TAPE
C  ND        FLUX INPUT OPTION
C  NDELK     NUMBER OF REACTIVITY COEFFICIENT CALCULATIONS(< OR = 20)
C  NFLUX1    FLUX TAPE
C  NFLUX2    ADJOINT FLUX TAPE
C  NFP       FLUX NORMALIZATION OPTION
C  NIBC      NUMBER OF ISOTOPES FOR BETA CALCULATION  (< OR = 20)
C  NINP      LOGICAL INPUT TAPE
C  NOUT      LOGICAL OUTPUT TAPE
C  NPRT      PRINT OPTION
C  NSCRAT    SCRATCH TAPE
C  POWER     TOTAL REACTOR POWER
C  RABS      ABSORPTION RATE
C  RL        RADIAL LEAKAGE
C  RLEAK     LEAKAGE RATE
C  RSORS     SOURCE RATE
C  TSD       (MW-SEC)/(FISSIONS)

```

C VF VOLUME FACTOR  
C ZKEFF KEFF FOR DELTA K CALC. (IF =0, USE KEFF CALC. BY CODE)  
C ZL AXIAL LEAKAGE  
C A(MCR,10) ID CARD FOR EACH MATERIAL CROSS SECTION SET  
C A0(IM) RADIAL AREA ELEMENT  
C A1(JM) AXIAL AREA ELEMENT  
C A2(JM) AREA AT I=IP BETWEEN J-1 AND J  
C ACT(IM,JM,NACT) ACTIVITY TRAVERSE  
C ADJ(IM,JM) ADJOINT FLUX  
C ADJR(IGM,IM) 1-D RADIAL ADJOINT FLUX  
C ADJZ(IGM,JM) 1-D AXIAL ADJOINT FLUX  
C ATW(MCR) MATERIAL ATOMIC WEIGHT  
C BETA(NIBC) ABSOLUTE NEUTRON YIELD PER DELAYED FISSION  
C C(ITL,IGM,MT) CROSS SECTION ARRAY  
C CO(ITL,MT) CROSS-SECTION ARRAY FOR CURRENT GROUP  
C CHI(IGM) FISSION SPECTRUM  
C CHIP(IGM) DELAYED NEUTRON FISSION SPECTRUM  
C CX(ITL,IGM) CROSS SECTION ARRAY FOR MATDK (EQUIVALENCED)  
C DUM(IMJM) DUMMY VARIABLE  
C HOLN(MCR) MATERIAL NAME  
C IO(M01) MIX NUMBERS  
C I1(M01) MATERIAL NUMBERS FOR MIX  
C I2(M01) MATERIAL DENSITIES FOR MIX  
C ID(12) IDENTIFICATION CARD ARRAY  
C L2(MT) ZONE NUMBER BY MATERIAL  
C LDUM(IMJM) DUMMY VARIABLE  
C M0(IM,JM) ZONE NUMBERS  
C M2(IZM) MATERIAL NUMBERS BY ZONE  
C MA(NACT) MATERIAL NUMBER FOR ACTIVITY TRAVERSE  
C MASS(IZM,MT) MASS OF MATERIAL IN ZONE  
C MATDK(NDELK) MATERIAL NUMBER FOR REACTIVITY COEFFICIENT CALCULATION  
C NBET(NIBC) MATERIAL NUMBER FOR BETA CALCULATION  
C NCOL(NDELK) COLUMN NUMBER FOR REACTIVITY COEFFICIENT CALCULATION  
C NROW(NDELK) ROW NUMBER FOR REACTIVITY COEFFICIENT CALCULATION  
C NX(NACT) CROSS SECTION POSITION FOR ACTIVITY TRAVERSE  
C PHC(IGM) CENTER POINT FLUX  
C PHI(IM,JM) FLUX  
C PHR(IGM,IM) 1-D RADIAL FLUX  
C PHZ(IGM,JM) 1-D AXIAL FLUX  
C R1(IP) RADII VALUES  
C R4(IM) AVERAGE RADII  
C R5(IM) DELTA-R  
C SLOAR(IM) D(ADJOINT)/DR (EQUIVALENCED)  
C SLOAZ(JM) D(ADJOINT)/DZ (EQUIVALENCED)  
C SLOPR(IM) D(FLUX)/DR  
C SLOPZ(JM) D(FLUX)/DZ  
C SORC1(IM,JM) NU\*SIGF\*FLUX (IN NORM)



```

C  SORC2(IM,JM) =CADJ(I,J)  CHI*ADJOINT  (IN NORM)
C  TPHI(IM,JM) TOTAL FLUX (EQUIVALENCED)
C  VO(IM,JM)  VOLUME ELEMENTS
C  VEL(IGM)  NEUTRON VELOCITY
C  VOL(IZM)  ZONE VOLUME
C  Z1(JP)  AXII VALUES
C  Z4(JM)  AVERAGE AXII
C  Z5(JM)  DELTA-Z
C  ZACT(IZM,NACT)  ZONE AVERAGED ACTIVITIES
C  ZPHI(IZM)  ZONE AVERAGED FLUX
C  ZPOW(IZM)  ZONE AVERAGED POWER
      DIMENSION JLPTAB(28)
      CALL SETIO(9,2)
      CALL SETIO(10,1)
      CALL SETDR(3,525000,50000,JLPTAB)
      CALL SETDR(4,575000,200000,JLPTAB(8))
      CALL SETDR(6,775000,200000,JLPTAB(15))
      CALL SETDR(7, 975000, 200000, JLPTAB(22))
      REWIND 3
      REWIND 4
      REWIND 6
      REWIND 7
      NCR1 = 3
      NSCRAT = 4
      NFLUX1 = 6
      NFLUX2 = 7
      NOUT = 9
      NINP = 10
      CALL INP1
      CALL MAPR(DUM)
      CALL SETUP
      CALL GRAM
      CALL NORM
      IF(NIBC) 8, 8, 6
6     CALL ABETA
8     CONTINUE
      IF(NDELK) 20, 20, 10
10    CALL CALC
20    CONTINUE
      END

```

```

-ITC FOR INP1,INP1
  SUBROUTINE INP1
    INCLUDE ABC
  C THIS SUBROUTINE CONTROLS THE READING OF ALL INPUT DATA
    WRITE(NOUT, 3)
  3   FORMAT(42X,35H * * * * P E R T - I V * * * * //)
    READ(NINP, 5) (ID(I), I = 1, 12)
  5   FORMAT(12A6)
    WRITE(NOUT, 5) (ID(I), I = 1, 12)
    WRITE(NOUT, 6)
  6   FORMAT(1H /)
    READ(NINP,10) ND, IGM, IST, IFP, IM, JM, MCR, MTP, IZM, MT,
  1   M01, NPRT, NDELK, NIBC, NACT,NFP, IGE
  10  FORMAT(12I6)
    READ(NINP,15) FLPO,VF,FEF, ZKEFF
  15  FORMAT(6E12.6)
    WRITE(NOUT, 20) ND, IGM, IST, IFP,IM, JM
  20  FORMAT (76H ND 1/2/3/4/5/6/7=FLUXES FROM 1-D RUNS/FLUXES F
  1ROM 2-D RUN/FLUX FROM/,12X,66H1-D,ADJOINT FROM 2-D/FLUX FROM 2-D,A
  2DJOINT FROM 1-D/FLUX FROM 1-D,/12X,56HNO ADJOINT/FLUX FROM 2-D,NO
  3ADJOINT/2-D FLUXES FROM TAPE,13X,I9/
  4   35H IGM NUMBER OF ENERGY GROUPS,46X,I9/,42H
  5IST NUMBER OF DOWNSCATTERING TERMS,39X,I9/,
  661H IFP 0/1/2=NO EFFECT/PUNCH FLUXES/PUNCH ADJOINT FLUXES,
  720X,I9/, 38H IM NUMBER OF RADIAL
  8INTERVALS,43X,I9/,37H JM NUMBER OF AXIAL INTERVALS,44X,I9)
  30  WRITE(NOUT, 30) MCR, MTP, IZM, MT, M01, NPRT
    FORMAT (42H MCR NUMBER OF MATERIALS FROM CARDS,39X,I9/,41H
  1 MTP NUMBER OF MATERIALS FROM TAPE,40X,I9/,36H IZM NU
  2MBER OF MATERIAL ZONES,45X,I9/,53H MT TOTAL NUMBER OF MATE
  3RIALS INCLUDING MIXES,28X,I9/,44H M01 NUMBER OF MIXTURE SPE
  4CIFICATIONS,37X,I9/,57H NPRT PRINT OPTION )0/1=PARTIAL PRI
  5NT/FULL PRINT*24X,I9/)
  40  WRITE(NOUT, 40) NDELK, NIBC, NACT
    FORMAT (66H NDELK 0/N=NO EFFECT/DO N REACTIVITY COEFFICIENT C
  1ALCULATIONS,15X,I9/
  272H NIBC 0/N=NO EFFECT/USE N MATERIALS FOR BETA EFFECTIVE CA
  3LCULATION, 9X,I9/
  449H NACT 0/N=NO EFFECT/DO N ACTIVITY TRAVERSES,32X,I9)
    WRITE(NOUT, 50) NFP
  50  FORMAT (78H NFP FLUX OR POWER NORMALIZATION )1/2/3/4=CENT
  1ER PT. FLUX/CENTER PT. /,12X,46HPOWER/TOTAL POWER/DENOMINATOR OF P
  2ERT FORMULA*,23X,I9 )
    WRITE(NOUT,55) IGE
  55  FORMAT( 46H IGE GEOMETRY )0/1/2=R-Z/X-Y/R-Theta*,35X,I9/)
    WRITE(NOUT, 60) FLPO,VF,FEF, ZKEFF
  60  FORMAT (41H FLPO FLUX(N/CM2-SEC) OR POWER(MWT),39X,1PE10.4/4

```

```

17H   VF      VOLUME FACTOR           ,33X,1PE10.4/
244H  FEF      ENERGY(MEV) RELEASED PER FISSION,36X,1PE10.4/
374H  ZKEFF    KEFF FOR DELTA K CALC.  )IF ZERO, USE KEFF CALCULAT
4ED BY CODE*, 6X, 1PE10.4)
64    FORMAT(1H1, 12A6///)
      ITL = IST + 5
C     READ, CHECK, PRINT, AND WRITE CROSS SECTIONS TO TAPE
      WRITE(NOUT, 64) (ID(I), I = 1, 12)
      CALL XSINP
C     READ FLUXES
      WRITE(NOUT, 64) (ID(I), I = 1, 12)
      WRITE(NOUT, 82)
82    FORMAT(98H INPUT FLUXES (PHR/PHZ/ADJR/ADJZ/PHI/ADJ=RADIAL FLUX/AXI
1AL FLUX/RADIAL ADJOINT FLUX/AXIAL ADJOINT /
232H FLUX/2-D FLUX/2-D ADJOINT FLUX)/)
      CALL FXINP
      IP=IM + 1
      JP=JM + 1
C     READ RADIIR
      WRITE(NOUT, 84)
84    FORMAT(51HOMESH BOUNDARIES (R1/Z1=RADIAL POINTS/AXIAL POINTS))
      CALL REAG2(6H R1,R1,IP)
C     READ AXII
      CALL REAG2(6H Z1,Z1,JP)
C     READ ZONE NUMBERS
      WRITE(NOUT,86)
86    FORMAT(30H0ZONE NUMBERS BY MESH INTERVAL)
      CALL REAI2 (6H M0,LDUM,IMJM)
      L = 0
      DO 87 J = 1, JM
      DO 87 I = 1, IM
      L = L + 1
87    MO(I,J) = LDUM(L)
C     READ MATERIAL NUMBERS
      WRITE(NOUT, 88)
88    FORMAT(25HOMATERIAL NUMBERS BY ZONE)
      CALL REAI2 (6H M2,M2,IZM)
C     READ FISSION FRACTIONS
      WRITE(NOUT, 90)
90    FORMAT(17HOFISSION SPECTRUM)
      CALL REAG2 (6H CHI,CHI,IGM)
C     READ MIXTURE SPECIFICATIONS
      WRITE(NOUT,92)
92    FORMAT(82HOMIXTURE SPECIFICATIONS (IO/I1/I2=MIX NUMBER/MAT. NUMBER
1 FOR MIX/MATERIAL DENSITY))
      CALL REAI2 (6H IO,IO,M01)
      CALL REAI2 (6H I1,I1,M01)

```

```

      CALL REAG2(6H I2,I2,M01)
C CHECK FOR LIFETIME AND BETA CALCULATIONS
  IF(NIBC) 120,120,100
C READ MATERIAL NUMBERS FOR BETAS
100 WRITE(NOUT,94)
94  FORMAT(38H MATERIAL NUMBERS FOR BETA CALCULATION)
    CALL REAI2(6H NBET,NBET,NIBC)
C READ BETAS
    CALL REAG2(6H BETA,BETA,NIBC)
C READ DELAYED FISSION SPECTRUM
    WRITE(NOUT,102)
102 FORMAT(25H DELAYED FISSION SPECTRUM)
    CALL REAG2(6H CHIP,CHIP,IGM)
C READ VELOCITIES
    WRITE(NOUT,104)
104 FORMAT(17H NEUTRON VELOCITY)
    CALL REAG2(6H VEL,VEL,IGM)
C CHECK FOR DELTA K CALCULATION
120 IF(NDELK) 160,160,140
C READ MATERIAL NUMBERS, ROW NUMBERS, AND COLUMN NUMBERS FOR
C PERTURBATION CALCULATION
140 WRITE(NOUT,138)
138 FORMAT(98H PERTURBATION CALCULATION INFORMATION (MATDK/NROW/NCOL=M
    MATERIAL NUMBER/ROW/COLUMN TO BE PERTURBED))
    CALL REAI2(6H MATDK, MATDK,NDELK)
    CALL REAI2(6H NROW, NROW, NDELK)
    CALL REAI2(6H NCOL, NCOL, NDELK)
C CHECK FOR ACTIVITIES
160 IF(NACT) 1000,1000,180
C READ MATERIAL NUMBERS AND XS POSITION FOR ACTIVITIES
180 WRITE(NOUT,178)
178 FORMAT(77H ACTIVITY TRAVERSE INFORMATION (MA/NX=MATERIAL NUMBER/CR
    LOSS SECTION POSITION))
    CALL REAI2(6H MA,MA,NACT)
    CALL REAI2(6H NX, NX, NACT)
1000 CONTINUE
    RETURN
    END

```

```

-ITC FOR XSINP,XSINP
  SUBROUTINE XSINP
    INCLUDE ABC
  C THIS SUBROUTINE READS, CHECKS, PRINTS, AND WRITES THE CROSS SECTIONS TO TAPE
    ITEMP = MCR + MTP
  10  DO 50 I=1,ITEMP
      READ(NINP, 20) HOLN(I), ATW(I), (A(I,J), J=1, 10)
  20  FORMAT(A6, E6.2, 10A6)
      IF(MCR) 35,35,25
  25  DO 40 IIG=1,IGM
      READ(NINP,30) (C(L,IIG,I), L=1,ITL)
  30  FORMAT(6E12.5)
      GO TO 50
  35  READ(15) ((C(L,IIG,I), L=1,ITL), IIG=1,IGM)
  50  CONTINUE
  C CHECK ON CROSS SECTION CONSISTENCY AND ORDER
      DO 200 J=1,ITEMP
      DO 200 I=1,IGM
      G = C(2,I,J) + C(5,I,J)
      DO 80 K=1,IST
      KK = I + K
      M = 5 + K
      IF(KK - IGM) 70, 70, 80
  70  G = G + C(M,KK,J)
  80  CONTINUE
      IF(ABS((G - C(4,I,J))/C(4,I,J)) - .01) 200, 90, 90
  90  WRITE(NOUT,110) J,I
  110 FORMAT(1H/,16H CHECK MATERIAL I2,5X, 7H GROUP I2)
  200 CONTINUE
  C PRINT CROSS SECTIONS
      IF(NPRT) 210,210, 250
  210 WRITE(NOUT, 220)
  220 FORMAT(38H THE FOLLOWING NUCLIDES ARE READ-IN //)
      DO 240 K=1,ITEMP
  240 WRITE(NOUT, 245) K, HOLN(K), (A(K,J), J=1,10)
  245 FORMAT(I9,6X,11A6/)
      GO TO 510
  250 DO 480 K=1,ITEMP
  255 FORMAT(1H1)
      WRITE(NOUT, 245) K, HOLN(K), (A(K,J), J=1,10)
      WRITE(NOUT,260)
  260 FORMAT( 1H0,
2113H GROUP SIGF SIGA NUSIGF SIGTR SIGXG G-1X
3G G-2XG G-3XG G-4XG G-5XG G-6XG/ )
      IF(IST - 6) 290, 290, 265
  265 DO 270 I=1,IGM
  268 FORMAT(I3,11F10.3)

```

```

270  WRITE(NOUT,268) I,(C(J,I,K),J=1,11)
      WRITE(NOUT,275)
275  FORMAT (44H1GROUP  G-7XG      G-8XG      G-9XG      G-10XG/ )
      DO 279 I=1,IGM
279  WRITE(NOUT,268) I,(C(J,I,K), J=12,ITL)
      GO TO 350
290  CONTINUE
      DO 300 I=1,IGM
300  WRITE(NOUT,268) I, (C(J,I,K), J=1,ITL )
350  IF(K -ITEMP) 460,480,480
460  WRITE(NOUT,255)
480  CONTINUE
C  WRITE CROSS SECTIONS TO TAPE
510  DO 1160 IIG=1,IGM
      DO 1060 M=1,MT
      DO 1060 L=1,ITL
1060  CO (L,M) = C(L,IIG,M)
1160  WRITE (NCR1) ((CO (L,M), L=1,ITL),M=1,MT)
      REWIND NCR1
      RETURN
      END

```

```

-ITC FOR FXINP,FXINP
  SUBROUTINE FXINP
    INCLUDE ABC
    INTEGER TEMP
C THIS SUBROUTINE READS FLUXES(REGULAR FLUX AND ADJOINT FLUX) IF
C FLUXES ARE IN I-D FORM, THEY ARE COMBINED IN SUCH A WAY TO PUT
C THEM IN 2-D FORM
    IMJM = IM*JM
    JJR = IGM*IM
    JJZ = IGM*JM
    LLH = 1
    NND = ND
    ND = IABS(ND)
C LLH=1/2=REGULAR FLUX/ADJOINT FLUX
    GO TO (100, 800, 100, 800, 100, 800,800),ND
C FLUX IS IN I-D FORM
100 CONTINUE
    ZFX = 0.0
    RFX = 0.0
    GO TO (110, 120), LLH
110 CALL REAG2(6H PHR,DUM,JJR)
    GO TO 130
120 CALL REAG2(6H ADJR,DUM,JJR)
C IF NND IS POSITIVE, ADJOINT FLUXES ARE REVERSED BY GROUP
    IF(NND) 130, 130, 121
121 CONTINUE
    L = 0
    DO 126 K = 1, IGM
    DO 124 I = 1, IM
    L = L + 1
    KK = IGM + 1 - K
124 PHR(KK, I) = DUM(L)
126 RFX = RFX + PHR(KK, I)
    GO TO 165
130 L = 0
    DO 160 K=1,IGM
    DO 158 I=1,IM
    L = L + 1
158 PHR(K,I) = DUM(L)
160 RFX = RFX + PHR(K,1)
165 CONTINUE
    GO TO (170, 180), LLH
170 CALL REAG2(6H PHZ,DUM,JJZ)
    GO TO 190
180 CALL REAG2(6H ADJZ,DUM,JJZ)
    IF(NND) 190, 190, 182
C SWITCH GROUPS

```

```

182 CONTINUE
    L = 0
    DO 186 K = 1, IGM
    DO 184 I = 1, JM
    L = L + 1
    KK = IGM + 1 - K
184 PHZ(KK, I) = DUM(L)
186 ZFX = ZFX + PHZ(KK, 1)
    GO TO 262
190 L = 0
    DO 260 K=1,IGM
    DO 258 I=1,JM
    L = L + 1
258 PHZ(K,I) = DUM(L)
260 ZFX = ZFX + PHZ(K,1)
262 CONTINUE
    DO 300 K=1,IGM
    DO 280 I=1,IM
280 PHR(K,I) = PHR(K,I)/RFX
    DO 300 I=1,JM
300 PHZ(K,I) = PHZ(K,I)/ZFX
    DO 400 K=1,IGM
    PHC(K) = (PHZ(K,1) + PHR(K,1))*(.5)
    DO 320 I=2,IM
320 PHR(K,I) = PHR(K,I)/PHR(K,1)
    DO 360 I=2,JM
360 PHZ(K,I) = PHZ(K,I)/PHZ(K,1)
    PHR(K,1) = 1.0
    PHZ(K,1) = 1.0
    DO 380 I=1,IM
    DO 380 J=1,JM
    PHI(I,J) = PHR(K,I)*PHZ(K,J)*PHC(K)
380 CONTINUE
    WRITE(NFLUX1) ((PHI(I,J), I = 1, IM), J = 1, JM)
400 CONTINUE
    GO TO 1000
790 CALL NTRAN (14,8,1)
C FLUX IS IN 2-D FORM
800 CONTINUE
    LWH = IGM*IMJM
    GO TO (810, 815), LLH
810 WRITE(NOUT, 812) LWH
812 FORMAT(6X, 6H PHI, I6)
    GO TO 817
815 WRITE(NOUT, 816) LWH
816 FORMAT(6X, 6H ADJ, I6)
818 FORMAT(10E12.5)

```



```

817 DO 840 K = 1, IGM
    IF (ND-6) 819,819,822
819 READ(NINP, 820) ((PHI(I, J), I = 1, IM), J = 1, JM)
820 FORMAT(6E12.6)
    GO TO 825
822 READ (14) ((PHI(I,J),I=1,IM),J=1,JM)
825 WRITE(NOUT,818) ((PHI(I, J), I = 1, IM), J = 1, JM)
    WRITE(NFLUX1)((PHI(I,J),I=1,IM),J=1,JM)
840 CONTINUE
    GO TO (910, 850), LLH
850 CONTINUE
C IF NND IS POSITIVE, ADJOINT FLUXES ARE REVERSED BY GROUP
  IF(NND) 910, 910, 860
860 CONTINUE
    REWIND NFLUX1
    DO 880 K = 1, IGM
      READ(NFLUX1)
880 CONTINUE
    DO 900 K = 1, IGM
      BACKSPACE NFLUX1
      READ(NFLUX1) ((PHI(I, J), I = 1, IM), J = 1, JM)
      WRITE(NSCRAT) ((PHI(I, J), I = 1, IM), J = 1, JM)
      BACKSPACE NFLUX1
900 CONTINUE
      REWIND NSCRAT
      REWIND NFLUX1
      TEMP = NSCRAT
      NSCRAT = NFLUX1
      NFLUX1 = TEMP
910 CONTINUE
1000 GO TO (1100, 2200), LLH
1100 LLH=2
C SWITCH TAPE DESIGNATIONS
  TEMP = NFLUX1
  NFLUX1 = NFLUX2
  NFLUX2 = TEMP
C READ ADJOINT FLUX
  GO TO (100, 800, 800, 100, 2000, 2000, 790), ND
C THERE IS NO ADJOINT FLUX INPUT
2000 CONTINUE
  AAA = 1.0
  DO 2100 K = 1, IGM
    WRITE(NFLUX1) ((AAA, I = 1, IM), J = 1, JM)
2100 CONTINUE
2200 REWIND NFLUX1
    REWIND NFLUX2
    IF(ND-6) 2220,2220,2210

```

```
2210 REWIND 14
C SWITCH TAPE DESIGNATIONS
2220 TEMP = NFLUX2
      NFLUX2 = NFLUX1
      NFLUX1 = TEMP
      RETURN
      END
```

```

-IT  FOR REAI2,REAI2
      SUBROUTINE REAI2(HOLL,IARRAY,NCOUNT)
      DIMENSION IARRAY(10),IV(6),K(6),IN(6)
      COMMON  NINP, NOUT, NCR1, NFLUX1, NSCRAT, NFLUX2
      J=1
      10 READ(NINP,20)      (K(I),IN(I),IV(I),I=1,6)
      20 FORMAT(6(I1,I2,I9))
      DO 70 I=1,6
      L=K(I)+1
      GO TO (30,40,60,80 ),L
CNO MODIFICATION
      30 IARRAY(J)=IV(I)
      J=J+1
      GO TO 70
C
CREPEAT
      40 L=IN(I)
      DO50M=1,L
      IARRAY(J)=IV(I)
      J=J+1
      50 CONTINUE
      GO TO70
C
CINTERPOLATE
      60 CALL ERRO2(6H REAI,60,1)
C
C
      70 CONTINUE
      GO TO 10
C
C
CTERMINATE
      80 J=J-1
      WRITE (NOUT,90)      HOLL,J      ,(IARRAY(I),I=1,J)
      IF(J -NCOUNT)100,110,100
      90 FORMAT(6X,A6,I6/(10I12))
      100 CALL ERRO2( 6H**REAI,100,1)
      110 RETURN
      END

```

```

-IT FOR REAG2,REAG2
  SUBROUTINE REAG2(HOLL,ARRAY,NCOUNT)
  DIMENSION ARRAY(10),V(12),K(12),IN(12)
  COMMON NINP, NOUT, NCR1, NFLUX1, NSCRAT, NFLUX2
  JFLAG=0
  J=1
  10 IF(JFLAG)20,40,20
  20 DO 30 JJ=1,6
    K(JJ)=K(JJ+6)
    IN(JJ)=IN(JJ+6)
  30 V(JJ)=V(JJ+6)
    JFLAG=0
    GO TO 60
  40 READ (NINP,50) (K(I),IN(I),V(I),I=1,6)
  50 FORMAT(6(I1,I2,E9.4))
  60 DO 140 I=1,6
    L=K(I)+1
    GO TO (70,80,100,150),L
CNO MODIFICATION
  70 ARRAY(J)=V(I)
    J=J+1
    GO TO 140
C
CREPEAT
  80 L=IN(I)
    DO90M=1,L
    ARRAY(J)=V(I)
    J=J+1
  90 CONTINUE
    GO TO140
C
CINTERPOLATE
  100 IF(I-6) 120,110,110
  110 READ (NINP,50) (K(JJ),IN(JJ),V(JJ),JJ=7,12)
    JFLAG=1
  120 L=IN(I)+1
    DEL=(V(I+1)-V(I))/FLOAT (L)
    DO 130 M=1,L
    ARRAY(J)=V(I)+DEL*FLOAT (M-1)
    J=J+1
  130 CONTINUE
C
C
  140 CONTINUE
    GO TO 10
C
C

```

```
CTERMINATE
150 J=J-1
    WRITE (NOUT,160)      HOLL,J      ,( ARRAY(I),I=1,J)
    IF(J -NCOUNT)170,180,170
160 FORMAT(6X,A6,I6/(10E12.5))
170 CALL ERRO2( 6H**REAG,170,1)
180 RETURN
    END
```

```

-ITC FOR MAPR,MAPR
  SUBROUTINE MAPR(K)
  INCLUDE ABC
  DIMENSION K(9)
  1 IX=IM
  IY=JM
  WRITE (NOUT,6) (ID(I), I = 1, 12)
  6 FORMAT(1H1,12A6//)
  3 DO 5 JJ=1,IY
  J=IY-JJ+1
  5 WRITE (NOUT,4) (MO(I,J),I=1,IX)
  4 FORMAT( 5H ,55I2)
  WRITE(NOUT,15)
  15 FORMAT(2H A/2H X/2H I/2H A/2H L//8H RADIAL)
  WRITE (NOUT,6) (ID(I), I = 1, 12)
  DO 10 JJ=1,IY
  J=IY-JJ+1
  DO 11 L=1,IX
  N=MO(L,J)
  11 K(L)=IABS (M2(N))
  10 WRITE (NOUT,4) (K(L),L=1,IX)
  WRITE(NOUT,15)
  RETURN
  END

```

```

-IT FOR PRT,PRT
  SUBROUTINE PRT(IM, JM, PHI)
  COMMON NINP, NOUT, NCR1, NFLUX1, NSCRAT, NFLUX2
  DIMENSION PHI(50, 50)
  DO 60 I=1,IM,5
  I1=I
  I2=I+4
  IF(I2-IM) 20 ,20 ,10
10 I2=IM
20 WRITE ( NOUT,30 ) ( JJ, JJ=I1, I2)
30 FORMAT( 5X, 5I20)
  DO 50 JJ=1, JM
  J=JJ
  WRITE(NOUT,40 ) J, (PHI(K,J), K=I1, I2)
40 FORMAT(5X, I5, E15.7, 4E20.7)
50 CONTINUE
60 CONTINUE
  RETURN
  END

```

```
-IT FOR ERRO2,ERRO2
  SUBROUTINE ERRO2( HOL,JSUBR,I)
  COMMON  NINP, NOUT, NCR1, NFLUX1, NSCRAT, NFLUX2
  WRITE (NOUT,1)      HOL,JSUBR
  1 FORMAT(2H */9H ERROR IN,A6.3H AT,I6/2H */2H *)
  GO TO (3,4),I
  3 STOP
  4 RETURN
  END
```



```

-ITC FOR SETUP, SETUP
  SUBROUTINE SETUP
    INCLUDE ABC
C   MIX CROSS SECTIONS
    ITL = IST + 5
    WRITE(NOUT, 65) (ID(I), I = 1, 12)
65   FORMAT(1H1,12A6///)
    WRITE (NOUT,70)
    70   FORMAT(/19H CROSS-SECTION EDIT)
        REWINDNCR1
        DO 100 IIG=1,IGM
        READ (NCR1) ((CO(I,J),I=1,ITL),J=1,MT)
        DO 92 M=1,M01
        IF(I0(M)-MT)74 ,74 ,72
    72   CALL ERRO2(6H**S807,72 ,1)
    74   IF(I1(M)-MT)80 ,72 ,72
    80   N=I0(M)
        L=I1(M)
        E01=I2(M)
        DO 90 I=1,ITL
        IF(L)86 ,88 ,86
    86   CO(I,N)=CO(I,N)+CO(I,L)*E01
        GOTO90
    88   CO(I,N)=CO(I,N)*E01
    90   CONTINUE
    92   CONTINUE
        WRITE (NOUT,94 ) IIG
    94   FORMAT( //7H GROUP I3,15H CROSS-SECTIONS/)
        DO 96 N=1,MT
    96   WRITE (NOUT,98 ) N,(CO(I,N),I=1,ITL)
    98   FORMAT(4H MAT,I3,10E11.3,/7X,10E11.3)
        WRITE (NSCRAT) ((CO(I,J),I=1,ITL),J=1,MT)
    100  CONTINUE
        REWINDNCR1
        REWIND NSCRAT
C   SWITCH TAPE DESIGNATIONS
    ITEMP=NSCRAT
    NSCRAT=NCR1
    NCR1=ITEMP
    101  FORMAT(1H1)
C   CALCULATE AREAS AND VOLUMES
C*****
C*****
    PI2=6.28318
    IGEP = IGE + 1
    DO 170 I=1,IM
    R4(I)=(R1(I+1)+R1(I))*0.5

```

```

R5(I)=R1(I+1)-R1(I)
IF( R5(I) ) 130, 130, 140
130 CALL ERRO2 (6H*R5(I),130 ,1)
140 CONTINUE
GO TO (160, 150, 150), IGEP
150 CONTINUE
A1(I)=R5(I)
GO TO 170
160 CONTINUE
A1(I)=PI2*R5(I)*R4(I)
170 CONTINUE
DO 200 J=1, JM
Z4(J)=(Z1(J+1)+Z1(J))*0.5
Z5(J)=Z1(J+1)-Z1(J)
IF(Z5(J)) 175, 175, 180
175 CALL ERRO2 (6H*Z5(J),175 ,1)
180 CONTINUE
GO TO (183, 182, 183), IGEP
182 A2(J) = Z5(J)
GO TO 184
183 A2(J) = PI2*R1(IP)*Z5(J)
184 DO 195 I = 1, IM
GO TO (190, 185, 190), IGEP
185 V0(I,J)=R5(I)*Z5(J)*VF
GO TO 195
190 V0(I,J)=PI2*R5(I)*Z5(J)*R4(I)*VF
195 CONTINUE
200 CONTINUE
C PRINT MIX INFORMATION AND RADII AND AXII INFORMATION
WRITE(NOUT,204) (ID(I), I = 1, 12)
204 FORMAT(1H1, 12A6//)
WRITE(NOUT, 208) (J, IO(J), I1(J), I2(J), J = 1, MO1)
208 FORMAT(3X, 16H MIXTURE NUMBER ,18H MIX COMMAND
124H MATERIAL ATOMIC DENSITY//(I3,1X,I8,8X,I8,8X,E20.8))
ZZ = 0.0
IP = IM + 1
JP = JM + 1
WRITE(NOUT, 65) (ID(I), I = 1, 12)
WRITE(NOUT, 210)
210 FORMAT(84H RADII AVG RADII
1 AXII AVG AXII /)
IF(IM - JM) 240, 240, 280
240 WRITE(NOUT, 220) ((I, R1(I), R4(I), Z1(I), Z4(I)), I = 1, IP)
220 FORMAT(I4,4F20.4)
IF(IM - JM) 241, 500, 241
241 III = IP + 1
WRITE(NOUT, 220) ((I, ZZ, ZZ, Z1(I), Z4(I)), I = III,JP)

```

```
GO TO 500
280 WRITE(NOUT, 220) ((I, R1(I), R4(I), Z1(I), Z4(I)), I = 1, JP)
    JJJ = JP + 1
500 WRITE(NOUT, 220) ((I, R1(I), R4(I), ZZ, ZZ), I = JJJ, IP)
    CONTINUE
    RETURN
    END
```

```

-ITC FOR GRAM,GRAM
SUBROUTINE GRAM
INCLUDE ABC
REAL MASS
DIMENSION MASS(50, 50)
EQUIVALENCE (C(1,1,1), MASS(1,1))
C THIS SUBROUTINE CALCULATES THE MASS OF THE VARIOUS MATERIALS
WRITE(NOUT, 1) (ID(I), I = 1, 12)
1 FORMAT(1H1,12A6///)
WRITE(NOUT, 5)
5 FORMAT(45H MATERIAL INVENTORY (KILOGRAMS) FOR EACH ZONE / )
DO 10 K=1,IZM
VOL(K) = 0.0
10 CONTINUE
DO 15 J = 1, 50
DO 15 I = 1, 50
15 MASS(I, J) = 0.0
DO 20 J = 1, JM
DO 20 I = 1, IM
K = M0(I,J)
KK = M2(K)
L2(KK) = K
VOL(K) = VOL(K) + V0(I, J)*.001
20 CONTINUE
KK = MCR + MTP
DO 40 N = 1, IZM
NN = M2(N)
DO 40 M = 1, M01
IF(I0(M) - NN) 40,22,40
22 L = I1(M)
IF(L - KK) 28,28,24
24 NNAA = L
DO 35 MAA = 1, M01
IF(I0(MAA) - NNAA) 35,32,35
32 L = I1(MAA)
IF(L) 35,35,33
33 E01 = I2(MAA)*I2(M)
MASS(N,L) = ((E01*ATW(L)*VOL(N))/ .6023) + MASS(N,L)
35 CONTINUE
GO TO 40
28 IF(L) 40,40,30
30 E01 = I2(M)
MASS(N,L) = ((E01*ATW(L)*VOL(N))/ .6023) + MASS(N,L)
40 CONTINUE
DATA ZONE/6H ZONE /
DO 90 L = 1, IZM, 5
LL = L + 4

```

```
IF(LL - IZM) 62, 62, 61
61 LL = IZM
62 WRITE(NOUT, 72) ((ZONE, K), K=L, LL)
72 FORMAT(/ /26H MATERIAL ATOMIC WT. ,3X, 5(A6,I2,12X))
WRITE(NOUT, 73) (VOL(K), K = L, LL)
73 FORMAT(25X, 5(F8.2, 7H LITERS, 5X))
DO 75 K = 1, KK
75 WRITE(NOUT, 76) K, HOLN(K), ATW(K), (MASS(I, K), I = L, LL)
76 FORMAT(/I3,1X, A6, F13.3, F14.3, 4F20.3)
IF(LL - IZM) 90, 94, 94
90 CONTINUE
94 CONTINUE
RETURN
END
```

```

-ITC FOR NORM,NORM
  SUBROUTINE NORM
  INCLUDE ABC
  DIMENSION TPHI(50, 50), CADJ(50, 50), ACT(50, 50, 3), POW(50, 50)
  EQUIVALENCE (SORC1(1,1),TPHI(1,1)), (SORC2(1,1),CADJ(1,1)),
  1(C(1,1,1),ACT(1,1,1)),(LDUM(1),POW(1,1))
C   THIS SUBROUTINE NORMALIZES THE FLUXES (REGULAR AND ADJOINT)
  CFX = 0.0
  CPW = 0.0
  POWER = 0.0
  DENOM = 0.0
  RL = 0.0
  ZL = 0.0
  RABS = 0.0
  RSORS = 0.0
  DO 10 J = 1, JM
  DO 10 I = 1, IM
  SORC1(I, J) = 0.0
  SORC2(I, J) = 0.0
10  CONTINUE
  REWIND NFLUX1
  REWIND NFLUX2
  REWIND NSCRAT
  REWIND NCR1
  DO 100 KK = 1, IGM
  READ(NCR1) ((C0(II, J), II = 1, ITL), J = 1, MT)
  READ(NFLUX1) ((PHI(I, J), I = 1, IM), J = 1, JM)
  READ(NFLUX2) ((ADJ(I, J), I = 1, IM), J = 1, JM)
  CFX = CFX + PHI(1, 1)
  K = M0(1, 1)
  K = M2(K)
  CPW = CPW + C0(1,K)*PHI(1,1)
  DO 20 J = 1, JM
  DO 20 I = 1, IM
  K = M0(I, J)
  K = M2(K)
  POWER = POWER + C0(1, K)*PHI(I, J)*V0(I, J)
  SORC1(I, J) = SORC1(I, J) + C0(3, K)*PHI(I, J)
  SORC2(I, J) = SORC2(I, J) + CHI(KK)*ADJ(I, J)
  RABS = RABS + C0(2, K)*PHI(I, J)*V0(I, J)
  RSORS = RSORS + C0(3, K)*PHI(I, J)*V0(I, J)
20  CONTINUE
  PI2 = 6.28318
  DO 50 I = 1, IM
  GO TO (35, 35, 25), IGEP
25  SLOPZ(I) = (PHI(I,JM) - PHI(I,JM - 1))/((Z4(JM) - Z4(JM - 1))
  1 *PI2*R4(I))

```

```

35 GO TO 40
   SLOPZ(I) = (PHI(I,JM) - PHI(I,JM - 1))/(Z4(JM) - Z4(JM - 1))
40 CONTINUE
   K = M0(I, JM)
   K = M2(K)
   ZL = ZL + (SLOPZ(I)*A1(I))/(3.*C0(4,K))
50 CONTINUE
   DO 60 I = 1, IM
   GO TO (55, 55, 52), IGEP
52 SLOPZ(I) = -(PHI(I,2) - PHI(I,1))/((Z4(2) - Z4(1))*PI2*R4(I))
   GO TO 56
55 SLOPZ(I) = -(PHI(I,2) - PHI(I,1))/(Z4(2) - Z4(1))
56 CONTINUE
   K = M0(I,1)
   K = M2(K)
   ZL = ZL + (SLOPZ(I)*A1(I))/(3.*C0(4,K))
60 CONTINUE
   DO 80 J=1, JM
   SLOPR(J) = (PHI(IM,J) - PHI(IM - 1,J))/(R4(IM) - R4(IM - 1))
   K = M0(IM,J)
   K = M2(K)
   RL = RL + (SLOPR(J)*A2(J))/(3.*C0(4,K))
80 CONTINUE
100 CONTINUE
   RLEAK = -ZL - RL
   RSORS = RSORS/VF
   RABS = RABS/VF
   AK = (RSORS)/(RABS + RLEAK)
C TSD = (MW-SEC)/(FISSIONS)
   TSD = FEF*1.602*10.**(-19)
   CAD = SORC2(1, 1)
   CPW = CPW*TSD*1000.
   POWER = POWER*TSD
   REWIND NCR1
   REWIND NFLUX1
   REWIND NFLUX2
   DO 200 J = 1, JM
   DO 200 I = 1, IM
   DENOM = DENOM + (SORC1(I, J) * SORC2(I, J)*V0(I, J))/(SORC2(1,1)*AK)
200 CONTINUE
   GO TO (240, 260, 280, 300), NFP
240 CONST = FLPO/CFX
   GO TO 320
260 CONST = FLPO/CPW
   GO TO 320
280 CONST = FLPO/POWER
   GO TO 320

```

```

300  CONST = FLPO/DENOM
320  CONTINUE
      CFX = CONST*CFX
      CPW = CONST*CPW
      POWER = CONST*POWER
      DENOM = CONST*DENOM
      WRITE(NOUT, 328) (ID(IK), IK = 1, 12)
328  FORMAT(1H1, 12A6////////)
      RSORS = RSORS*VF*CONST
      RABS = RABS*VF*CONST
      RLEAK = RLEAK*VF*CONST
      RL = -RL*VF*CONST
      ZL = -ZL*VF*CONST
      AAK = .1*AK
      WRITE(NOUT, 330) CFX, CPW, POWER, DENOM,      RSORS, RABS, RLEAK
1    , RL, ZL, AAK
330  FORMAT(44H CENTER POINT FLUX (N/CM2-SEC) - - - 1PE12.5//
1    44H CENTER POINT POWER (MWT/LITER) - - - 1PE12.5//
2    44H TOTAL POWER (MWT) - - - 1PE12.5//
3    44H DENOMINATOR OF PERT. FORMULA - - - 1PE12.5//
4    44H SOURCE RATE - - - 1PE12.5//
5    44H ABSORPTION RATE - - - 1PE12.5//
6    44H LEAKAGE RATE (TOTAL) - - - 1PE12.5//
7    44H RADIAL LEAKAGE - - - 1PE12.5//
8    44H AXIAL LEAKAGE - - - 1PE12.5//
9    44H KEFF - - - F12.5)
      DO 350 J = 1, JM
      DO 350 I = 1, IM
344  DO 344 N = 1, 3
      ACT(I, J, N) = 0.0
      TPhi(I, J) = 0.0
      CADJ(I, J) = 0.0
      POW(I, J) = 0.0
350  CONTINUE
      DO 400 K = 1, IGM
      READ(NFLUX1) ((PHI(I, J), I = 1, IM), J = 1, JM)
      READ(NFLUX2) ((ADJ(I, J), I = 1, IM), J = 1, JM)
      READ(NCR1) ((CV(II, J), II = 1, ITL), J = 1, MT)
      DO 360 J = 1, JM
      DO 360 I = 1, IM
      KK = M0(I, J)
      KK = M2(KK)
      PHI(I, J) = PHI(I, J)*CONST
      ADJ(I, J) = ADJ(I, J)/CAD
      TPhi(I, J) = TPhi(I, J) + PHI(I, J)
      CADJ(I, J) = CADJ(I, J) + CHI(K) * ADJ(I, J)
      POW(I, J) = POW(I, J) + C0(1, KK)*PHI(I, J)*TSD*1000.

```



```

352 IF(NACT) 360, 360, 352
CONTINUE
DO 356 N = 1, NACT
KK = MA(N)
NN = NX(N)
356 ACT(I, J, N) = ACT(I, J, N) + CO(NN, KK)*PHI(I, J)
360 CONTINUE
WRITE(NSCRAT) ((PHI(I, J), I = 1, IM), J = 1, JM)
WRITE(NSCRAT) ((ADJ(I, J), I = 1, IM), J = 1, JM)
LMN = IFP + 1
GO TO (368, 362, 364), LMN
362 PUNCH 363, ((PHI(I, J), I = 1, IM), J = 1, JM)
363 FORMAT(6E12.6)
GO TO 368
364 PUNCH 363, ((ADJ(I, J), I = 1, IM), J = 1, JM)
368 IF(NPRT) 376, 376, 370
370 CONTINUE
WRITE(NOUT, 328) (ID(IK), IK = 1, 12)
WRITE(NOUT, 372) K
372 FORMAT(//15H FLUX FOR GROUP,I3/)
CALL PRT(IM, JM, PHI)
WRITE(NOUT, 328) (ID(IK), IK = 1, 12)
WRITE(NOUT, 374) K
374 FORMAT(//23H ADJOINT FLUX FOR GROUP,I3/)
CALL PRT(IM, JM, ADJ)
376 CONTINUE
400 CONTINUE
WRITE(NOUT, 328) (ID(IK), IK = 1, 12)
WRITE(NOUT, 478)
478 FORMAT(11H TOTAL FLUX/)
CALL PRT(IM, JM, TPHI)
WRITE(NOUT, 328) (ID(IK), IK = 1, 12)
WRITE(NOUT, 480)
480 FORMAT(28H FISSION SOURCE*ADJOINT FLUX/)
CALL PRT(IM, JM, CADJ)
WRITE(NOUT, 328) (ID(IK), IK = 1, 12)
WRITE(NOUT, 482)
482 FORMAT (26H POWER DENSITY (MWT/LITER) /)
CALL PRT(IM, JM, POW)
IF(NACT) 490, 490, 484
484 DO 488 N = 1, NACT
DO 486 J = 1, JM
DO 486 I = 1, IM
486 SORC2(I, J) = ACT(I, J, N)
WRITE(NOUT, 328) (ID(IK), IK = 1, 12)
KK = MA(N)
NN = NX(N)

```

```
487 WRITE(NOUT, 487) N, KK, NN
    FORMAT( 9H ACTIVITY, I3, 13H ISOTOPE, I3, 28H CROSS SECT I
    ION POSITION, I3/)
    CALL PRT(IM, JM, SORC2)
488 CONTINUE
490 CONTINUE
    CALL AVG(TPHI, POW, ACT)
C SWITCH TAPE DESIGNATIONS
  ITEMP = NSCRAT
  NSCRAT = NFLUX1
  NFLUX1 = ITEMP
  REWIND NSCRAT
  REWIND NFLUX1
  REWIND NFLUX2
  REWIND NCR1
  RETURN
  END
```

```

-ITC FOR AVG,AVG
  SUBROUTINE AVG(TPHI,POW,ACT)
  INCLUDE ABC
C   THIS SUBROUTINE CALCULATES ZONE AVERAGED FLUXES, POWERS,
C   AND ACTIVITIES
  DIMENSION ACT(50, 50, 3), TPHI(50,50), POW(50,50)
  DATA CAT/6H ACT /
  WRITE(NOUT,328) (ID(IK),IK=1,12)
328  FORMAT(1H1,12A6////////)
  DO 340 KKK=1,IZM
  ZPHI(KKK) = 0.0
  ZPOW(KKK) = 0.0
  DO 340 N=1,3
340  ZACT(KKK,N) = 0.0
  DO 525 J=1,JM
  DO 525 I=1,IM
  KKK = MO(I,J)
  ZPHI(KKK) = ZPHI(KKK) + VO(I,J)*.001*TPHI(I,J)
  ZPOW(KKK) = ZPOW(KKK) + VO(I,J)*.001*POW(I,J)
  IF(NACT) 525,525,505
  DO 508 N=1,NACT
508  ZACT(KKK,N) = ZACT(KKK,N) + VO(I,J)*.001*ACT(I,J,N)
525  CONTINUE
  DO 540 KKK=1,IZM
  ZPHI(KKK) = ZPHI(KKK)/VOL(KKK)
  ZPOW(KKK) = ZPOW(KKK)/VOL(KKK)
  IF(NACT) 540,540,535
535  DO 538 N=1, NACT
538  ZACT(KKK,N) = ZACT(KKK,N)/VOL(KKK)
540  CONTINUE
  IF(NACT) 560,560,580
560  WRITE(NOUT,565)
565  FORMAT( 48H ZONE AV FLUX AV POWER //)
  DO 570 KKK=1,IZM
570  WRITE(NOUT,575) KKK,ZPHI(KKK),ZPOW(KKK)
575  FORMAT(/16,5E20.7)
  GO TO 600
580  WRITE(NOUT,585) ((CAT,N), N=1,NACT)
585  FORMAT( 48H ZONE AV FLUX AV POWER 6X,
1 3(A6,I1,13X))
  DO 590 KKK=1,IZM
590  WRITE(NOUT,575) KKK, ZPHI(KKK),ZPOW(KKK), (ZACT(KKK,N),N=1,NACT)
600  CONTINUE
  RETURN
  END

```

```

-ITC FOR ABETA,ABETA
  SUBROUTINE ABETA
    INCLUDE ABC
    DIMENSION BSIGF(50,50)
    EQUIVALENCE (PHR(1,1), BSIGF(1,1))
C   THIS SUBROUTINE CALCULATES BETA EFFECTIVE AND NEUTRON LIFETIME
    WRITE(NOUT,5) (ID(I), I=1,12)
5   FORMAT(1H1,12A6////)
    DO 10 J=1,50
    DO 10 I=1,50
    SORC1(I,J)=0.0
    SORC2(I,J)=0.0
    PHZ(I,J) = 0.0
    BSIGF(I,J) = 0.0
10  CONTINUE
    DO 1000 KK=1,IGM
    READ(NFLUX1) ((PHI(I,J), I=1,IM), J=1,JM)
    READ(NFLUX1) ((ADJ(I,J), I=1,IM), J=1,JM)
    READ(NCR1) ((CO(II,J), II=1,ITL),J=1,MT)
    DO 500 J = 1, JM
    DO 500 I = 1, IM
    PHZ(I,J) = (PHI(I,J)*ADJ(I,J))/(VEL(KK)) + PHZ(I,J)
    KKK= M0(I,J)
    KKK = M2(KKK)
    DO 300 L = 1, NIBC
    DEN = 0.0
    DO 120 M=1, M01
    IF(I0(M) -KKK) 120, 110, 120
110  IF(I1(M) - NBET(L)) 120, 115, 120
115  DEN = DEN + I2(M)
120  CONTINUE
    NN = NBET(L)
    BSIGF(I,J) = BSIGF(I,J) + BETA(L)*DEN*CO(1,NN)*PHI(I,J)
300  CONTINUE
C   DELAYED NEUTRON SOURCE*ADJOINT FLUX (SUMMED OVER ENERGY GROUPS)
    SORC2(I,J) = CHIP(KK)*ADJ(I,J) + SORC2(I,J)
500  CONTINUE
1000 CONTINUE
    BE = 0.0
    ALIFE = 0.0
    DO 2000 J = 1, JM
    DO 2000 I = 1, IM
    BE = BE + SORC2(I,J)* BSIGF(I,J)*V0(I,J)
    ALIFE = ALIFE + (PHZ(I,J)*V0(I,J))
2000 CONTINUE
    ALIFE = ALIFE/(DENOM*AK)
    BEFF = BE/(DENOM*AK)

```

```
2050 WRITE(NOUT,2050) BEFF, ALIFE
      FORMAT( 43H BETA EFFECTIVE - - - - - 1PE11.5//
1       43H NEUTRON GENERATION TIME - - - - - 1PE11.5)
      RETURN
      END
```

```

-ITC FOR CALC,CALC
  SUBROUTINE CALC
  INCLUDE ABC
  DIMENSION CX(15,26), FLUX(2,26,50,3),          SLOAR(50),
1          SLOAZ(50)
  EQUIVALENCE (CX(1,1), PHR(1,1)), (FLUX(1,1,1,1),C(1,1,1)),
1          (SLOAR(1), DUM(51)),
2 (SLOAZ(1), DUM(151))
C THIS SUBROUTINE USES PERTURBATION THEORY TO CALCULATE REACTIVITY
C COEFFICIENTS
  DO 9999 III = 1, NDELK
  REWIND NCR1
  REWIND NFLUX1
  REWIND NFLUX2
  REWIND NSCRAT
  NR = NROW(III)
  NC = NCOL(III)
  MAT = MATDK(III)
C CONVERSION FROM ATOMS TO KG
  CT= 602.3/ATW(MAT)
  PI2 = 6.28318
  DO 1 J = 1, 50
  DO 1 I = 1, 50
  SORC1(I, J) = 0.0
1 SORC2(I, J) = 0.0
  WRITE(NOUT, 2) (ID(I), I = 1, 12)
2 FORMAT(1H1,12A6// )
  DO 1000 K = 1, IGM
  READ(NFLUX1) ((PHI(I, J), I = 1, IM), J = 1, JM)
  READ(NFLUX1) ((ADJ(I, J), I = 1, IM), J = 1, JM)
  READ(NCR1) ((C0(II, J), II = 1, ITL), J = 1, MT)
  DO 3 II = 1, ITL
3 CX(II, K) = C0(II, MAT)
  IF(NR) 400, 400, 10
C DELTA K CALCULATION FOR ROW = NROW
10 CONTINUE
  DO 60 I = 1, IM
  IF(NR - JM) 20, 40, 40
20 NN = NR + 1
  FLUX(1, K, I, 3) = PHI(I, NN)
  FLUX(2, K, I, 3) = ADJ(I, NN)
30 IF(NR - 1) 50, 50, 40
40 NN = NR - 1
  FLUX(1, K, I, 1) = PHI(I, NN)
  FLUX(2, K, I, 1) = ADJ(I, NN)
50 NN = NR
  FLUX(1, K, I, 2) = PHI(I, NN)

```

```

FLUX(2, K, I, 2) = ADJ(I, NN)
SORC2(I, 1) = CHI(K)*FLUX(2, K, I, 2) + SORC2(I, 1)
SORC1(I, 1) = C0(3, MAT)*FLUX(1, K, I, 2) + SORC1(I, 1)
SORC1(I, 2) = C0(2, MAT)*FLUX(1, K, I, 2)*FLUX(2, K, I, 2)
1 + SORC1(I, 2)
60 CONTINUE
DO 150 I=1,IM
IF(I-1) 82,82,72
72 PSR1 = (FLUX(1,K,I,2) - FLUX(1,K,I-1,2))/(R4(I) - R4(I-1))
ASR1 = (FLUX(2,K,I,2) - FLUX(2,K,I-1,2))/(R4(I) - R4(I-1))
76 IF(I-IM) 78,86,86
78 PSR2 = (FLUX(1,K,I+1,2) - FLUX(1,K,I,2))/(R4(I+1) - R4(I))
ASR2 = (FLUX(2,K,I+1,2) - FLUX(2,K,I,2))/(R4(I+1) - R4(I))
GO TO 90
82 SLOPR(I) = 0.0
SLOAR(I) = 0.0
GO TO 92
86 SLOPR(I) = PSR1
SLOAR(I) = ASR1
GO TO 92
90 SLOPR(I) =(PSR1 + PSR2)/2.0
SLOAR(I) =(ASR1 + ASR2)/2.0
92 CONTINUE
IF(NR - 1) 100, 100, 93
93 GO TO (96, 96, 94), IGEP
94 PSZ1 = (FLUX(1,K,I,2) - FLUX(1,K,I,1))/(Z4(NR) - Z4(NR-1))
1 *PI2*R4(I))
ASZ1 = (FLUX(2,K,I,2) - FLUX(2,K,I,1))/(Z4(NR) - Z4(NR-1))
1 *PI2*R4(I))
GO TO 97
96 PSZ1 = (FLUX(1,K,I,2) - FLUX(1,K,I,1))/(Z4(NR) - Z4(NR-1))
ASZ1 = (FLUX(2,K,I,2) - FLUX(2,K,I,1))/(Z4(NR) - Z4(NR-1))
97 IF(NR -JM) 100, 115, 115
100 GO TO (105, 105, 102), IGEP
102 PSZ2 = (FLUX(1,K,I,3) - FLUX(1,K,I,2))/(Z4(NR+1) - Z4(NR))
1 *PI2*R4(I))
ASZ2 = (FLUX(2,K,I,3) - FLUX(2,K,I,2))/(Z4(NR+1) - Z4(NR))
1 *PI2*R4(I))
GO TO 106
105 PSZ2 = (FLUX(1,K,I,3) - FLUX(1,K,I,2))/(Z4(NR+1) - Z4(NR))
ASZ2 = (FLUX(2,K,I,3) - FLUX(2,K,I,2))/(Z4(NR+1) - Z4(NR))
106 IF(NR-1) 108, 108, 120
108 SLOPZ(I) = PSZ2
SLOAZ(I) = ASZ2
GO TO 130
112 CONTINUE
IF(NR -JM) 120,115,115

```

```

115  SLOPZ(I) = PSZ1
      SLOAZ(I) = ASZ1
      GO TO 130
120  CONTINUE
      SLOPZ(I)=(PSZ1 + PSZ2)/2.0
      SLOAZ(I)=(ASZ1 + ASZ2)/2.0
130  CONTINUE
      KK = M0(I, NR)
      KK = M2(KK)
      SORC1(I,3) = -((SLOPZ(I)*SLOAZ(I) + SLOPR(I)*SLOAR(I))*C0(4,MAT))/
1    (C0(4,KK)*C0(4,KK)*3.0) + SORC1(I,3)
150  CONTINUE
      GO TO 1000
C    DELTA K CALCULATION FOR COLUMN = NCOL
400  CONTINUE
      DO 460 J=1,JM
      IF(NC - IM) 420,440,440
420  NN = NC + 1
      FLUX(1,K,J,3) = PHI(NN,J)
      FLUX(2,K,J,3) = ADJ(NN,J)
430  IF(NC - 1) 450,450,440
440  NN = NC - 1
      FLUX(1,K,J,1) = PHI(NN,J)
      FLUX(2,K,J,1) = ADJ(NN,J)
450  NN = NC
      FLUX(1,K,J,2) = PHI(NN,J)
      FLUX(2,K,J,2) = ADJ(NN,J)
      SORC2(J,1) = CHI(K)*FLUX(2,K,J,2) + SORC2(J,1)
      SORC1(J,1) = C0(3,MAT)*FLUX(1,K,J,2) + SORC1(J,1)
      SORC1(J,2) = C0(2,MAT)*FLUX(1,K,J,2)*FLUX(2,K,J,2)
1    + SORC1(J,2)
460  CONTINUE
      DO 550 J=1,JM
      IF(J - 1) 478, 478, 472
472  GO TO (474, 474, 473), IGEP
473  PSZ1 = (FLUX(1,K,J,2) - FLUX(1,K,J-1,2))/((Z4(J) - Z4(J-1))
1    *PI2*R4(NC))
      ASZ1 = (FLUX(2,K,J,2) - FLUX(2,K,J-1,2))/((Z4(J) - Z4(J-1))
1    *PI2*R4(NC))
      GO TO 476
474  PSZ1 = (FLUX(1,K,J,2) - FLUX(1,K,J-1,2))/(Z4(J) - Z4(J-1))
      ASZ1 = (FLUX(2,K,J,2) - FLUX(2,K,J-1,2))/(Z4(J) - Z4(J-1))
476  IF(J-JM) 478,486,486
478  GO TO (480, 480, 479), IGEP
479  PSZ2 = (FLUX(1,K,J+1,2) - FLUX(1,K,J,2))/((Z4(J+1) - Z4(J))
1    *PI2*R4(NC))
      ASZ2 = (FLUX(2,K,J+1,2) - FLUX(2,K,J,2))/((Z4(J+1) - Z4(J))

```



```

1 *P12*R4(NC)
GO TO 481
480 PSZ2 = (FLUX(1,K,J+1,2) - FLUX(1,K,J,2))/(Z4(J+1) - Z4(J))
ASZ2 = (FLUX(2,K,J+1,2) - FLUX(2,K,J,2))/(Z4(J+1) - Z4(J))
481 IF(J - 1) 482, 482, 490
482 SLOPZ(J) = PSZ2
SLOAZ(J) = ASZ2
GO TO 492
486 SLOPZ(J) = PSZ1
SLOAZ(J) = ASZ1
GO TO 492
490 SLOPZ(J) = (PSZ1 + PSZ2)/2.0
SLOAZ(J) = (ASZ1 + ASZ2)/2.0
492 CONTINUE
IF(NC - 1) 508, 508, 495
495 PSR1 = (FLUX(1,K,J,2) - FLUX(1,K,J,1))/(R4(NC) - R4(NC-1))
ASR1 = (FLUX(2,K,J,2) - FLUX(2,K,J,1))/(R4(NC) - R4(NC-1))
498 IF(NC - IM) 500, 515, 515
500 PSR2 = (FLUX(1,K,J,3) - FLUX(1,K,J,2))/(R4(NC+1) - R4(NC))
ASR2 = (FLUX(2,K,J,3) - FLUX(2,K,J,2))/(R4(NC+1) - R4(NC))
IF(NC - 1) 508, 508, 520
508 SLOPR(J) = 0.0
SLOAR(J) = 0.0
GO TO 530
512 CONTINUE
IF(NC - IM) 520,515,515
515 SLOPR(J) = PSR1
SLOAR(J) = ASR1
GO TO 530
520 CONTINUE
SLOPR(J) = (PSR1 + PSR2)/2.0
SLOAR(J) = (ASR1 + ASR2)/2.0
530 CONTINUE
KK = M0(NC,J)
KK = M2(KK)
SORC1(J,3) = -((SLOPZ(J)*SLOAZ(J) + SLOPR(J)*SLOAR(J))*C0(4,MAT))/
1 (C0(4,KK)*C0(4,KK)*3.0) + SORC1(J,3)
550 CONTINUE
1000 CONTINUE
IF(IGM - 1) 1210, 1210, 1010
1010 IGGM = IGM - 1
DO 1200 K = 1, IGGM
IF(NR) 1100, 1100, 1050
1050 DO 1075 I = 1, IM
DO 1070 NN = 1, IST
N = NN + K
L = 5 + NN

```

```

        SORC2(I, 2) = SORC2(I, 2) + FLUX(1, K, I, 2)*CX(L, N)*
1 (FLUX(2, K, I, 2) - FLUX(2, N, I, 2))
        IF(N - IGM) 1070, 1075, 1075
1070 CONTINUE
1075 CONTINUE
        GO TO 1200
1100 DO 1175 J = 1, JM
        DO 1170 NN = 1, IST
        N = NN + K
        L = 5 + NN
        SORC2(J, 2) = SORC2(J, 2) + FLUX(1, K, J, 2)*CX(L, N)*
1 (FLUX(2, K, J, 2) - FLUX(2, N, J, 2))
        IF(N - IGM) 1170, 1175, 1175
1170 CONTINUE
1175 CONTINUE
1200 CONTINUE
1210 CONTINUE
        IF(ZKEFF - .001) 1250, 1250, 1225
C SET KEFF EQUAL TO READ IN VALUE
1225 DENOM = DENOM*AK/ZKEFF
        AK = ZKEFF
1250 CONTINUE
        IF(NR) 3000, 3000, 2000
2000 WRITE(NOUT, 2050) MAT, HOLN(MAT), NR
        WRITE(NOUT, 2060)
        WRITE(NOUT, 2062)
        SUM = 0.0
        TOT = 0.0
        DO 2200 I = 1, IM
C FISSIONS
        SORC2(I, 3) = (SORC2(I, 1)*SORC1(I, 1)*CT)/(AK*DENOM)
C ABSORPTIONS
        SORC1(I, 2) = (SORC1(I, 2)*CT)/DENOM
C SLOWING DOWN
        SORC2(I, 2) = (SORC2(I, 2)*CT)/DENOM
C LEAKAGE
        SORC1(I, 3) = (SORC1(I, 3)*CT)/DENOM
C DELTA K
        SORC2(I, 4) = SORC2(I, 3) - SORC1(I, 2) - SORC1(I, 3) - SORC2(I, 2)
        SUM = SUM + SORC2(I, 4)*V0(I, NR)
        TOT = TOT + V0(I, NR)
        ST = SUM/TOT
2050 FORMAT( 38HOREACTIVITY COEFFICIENTS FOR MATERIAL I2, 6X, A6,
1 11H      ROW I2//)
2060 FORMAT(131HU      #K/K PER KG      #K      #K      #K
1      #K      #K      #K      AVG RADII
2  AVG AXII )

```

```

2062 FORMAT( 99H          (NUSIGF)          (SIGA)
1      (SIGTR)          (SIGJXK)          (INTG) / )
      SORC1(I,2) = -SORC1(I,2)
      SORC1(I,3) = - SORC1(I,3)
      SORC2(I,2) = -SORC2(I,2)
      WRITE(NOUT, 2080) I, SORC2(I,4), SORC2(I,3), SORC1(I,2),
1      SORC1(I,3), SORC2(I,2), ST, R4(I), Z4(NR)
2080 FORMAT(I3, 8F16.4)
2200 CONTINUE
      GO TO 3400
3000 WRITE(NOUT, 3050) MAT,HOLN(MAT), NC
      WRITE(NOUT, 2060)
      WRITE(NOUT, 2062)
      SUM = 0.0
      TOT = 0.0
      DO 3200 I = 1, JM
C     FISSIONS
      SORC2(I, 3) = (SORC2(I,1)*SORC1(I,1)*CT)/(AK*DENOM)
C     ABSORPTIONS
      SORC1(I, 2) = (SORC1(I, 2)*CT)/DENOM
C     SLOWING DOWN
      SORC2(I, 2) = (SORC2(I, 2)*CT)/DENOM
C     LEAKAGE
      SORC1(I, 3) = (SORC1(I, 3)*CT)/DENOM
C     DELTA K
      SORC2(I, 4) = SORC2(I,3) - SORC1(I,2) - SORC1(I,3) - SORC2(I,2)
      SUM = SUM + SORC2(I,4)*V0(NC,I)
      TOT = TOT + V0(NC,I)
      ST = SUM/TOT
3050 FORMAT( 38HOREACTIVITY COEFFICIENTS FOR MATERIAL I2, 6X,A6,
1 14H          COLUMN I2//)
      SORC1(I,2) = -SORC1(I,2)
      SORC1(I,3) = - SORC1(I,3)
      SORC2(I,2) = -SORC2(I,2)
      WRITE(NOUT, 2080) I, SORC2(I,4), SORC2(I,3), SORC1(I,2),
1      SORC1(I,3), SORC2(I,2),ST, R4(NC), Z4(I)
3200 CONTINUE
3400 CONTINUE
9999 CONTINUE
      REWIND NCR1
      REWIND NFLUX1
      REWIND NFLUX2
      REWIND NSCRAT
      RETURN
      END

```