

AN ABSTRACT OF THE THESIS OF

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Title: A Simulated Two-Group Method for Computing Boiling Water  
Reactor Power Distributions

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A computer code capable of modeling the burnup dependent static behavior of a boiling water reactor was developed. The code calculates three-dimensional quarter core power distributions using a simulated two-group diffusion theory model. Thermal-hydraulic calculations account for the interaction between power, coolant flow, and void fraction distributions. Cross sections are corrected to account for the effect of control rods and local equilibrium xenon concentrations. Burnup step calculations are used to predict the effects of fuel depletion. The code will optionally calculate a unique power distribution which will minimize power peaking throughout a specified operating cycle. With these capabilities, the code can be used to develop BWR fuel loading and operation strategies which can aid in minimizing fuel cycle costs.

A Simulated Two-Group Method for Computing  
Boiling Water Reactor Power Distributions

by

Ronald Leonard Hatteberg

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APPROVED:

\_\_\_\_\_  
Professor of Nuclear Engineering

\_\_\_\_\_  
Head of Department of Nuclear Engineering

\_\_\_\_\_  
Dean of Graduate School

Date thesis is presented December 5, 1977.

Typed by Delores Maneshi for Ronald Leonard Hatteberg.

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# A Simulated Two-Group Method for Computing Boiling Water Reactor Power Distributions

## 1.0 INTRODUCTION

Analysis of the burnup dependent static behavior of a reactor core plays an important part in reactor fuel management. The ability to predict this behavior enables core design and operations strategies to be developed which can be used to minimize fuel cycle costs. The purpose of this project was to develop a boiling water reactor (BWR) model which could be used for this type of analysis.

A FORTRAN computer code capable of predicting three-dimensional power distributions in a BWR was developed. The model represents one quarter of a reactor core. Quadrant symmetry is assumed. The mutual interaction between the core power, fuel exposure, coolant flow and void fraction, and equilibrium xenon distributions are accounted for. Control rod effects are also included. Only fuel regions can be represented by the code. Reflecting regions surrounding a core are accounted for by appropriate boundary conditions.

The global power distribution is solved by three successive levels of iteration. The inner iteration solves the neutron flux distribution for a fixed fission source distribution. The next level of iteration, the source iteration, calculates the fission source distribution and effective multiplication factor given the nuclear constants of the core. The final level of iteration solves for the power distribution and accounts for the feedback effects between power, void fraction, and equilibrium xenon distributions.

Fuel depletion effects are modeled by burnup step calculations.



During a burnup step the power distribution is assumed to remain constant. The exposure distribution at the end of a step is calculated directly from the initial exposure distribution and the power distribution. Relative power and exposure distribution arrays can be punched by the code to facilitate restart calculations.

Input to the code consists primarily of a description of the core geometry and fuel loading, core thermal-hydraulic parameters, and tables of exposure and void dependent nuclear constants of the fuel. Output includes the predicted core power distribution, effective multiplication factor, coolant flow and void fraction distributions, and the predicted exposure distribution at the end of each burnup step.

BWR's have a strongly negative local power coefficient of reactivity. This is because the reactor coolant is allowed to change phase. Voiding of the coolant causes a large decrease in its moderating ability. There is a natural tendency for the power distribution to peak near the bottom of the core due to the larger void fractions at the top. Increasing the coolant flow rate reduces void fractions. Regulation of the coolant flow provides one of the primary means of BWR reactivity control. Due to these effects, a thermal-hydraulic model is an essential part of a BWR simulator.

A BWR simulator must be able to calculate three-dimensional power distributions, with feedback effects included, using a minimal amount of computer time and memory in order to be practical for routine application. This precludes an accurate but costly fine mesh multi-group diffusion theory type of calculation. One approach, used by the FLARE<sup>1</sup> code, is a one group, coarse mesh model using transport kernels to cou-

ple the fluxes at mesh points. This is equivalent to a one-group diffusion theory calculation. Various versions of the FLARE code are currently in wide use, but the model has several drawbacks. It requires extensive a priori normalization.

The neutronics model chosen for this project is an extension of a two-dimensional model developed for PWR analysis by Stout<sup>2</sup>. A simulated two-group diffusion theory model is used. This is based on a one-fast-group 3DB<sup>3</sup> type of calculation. The model permits a coarse mesh representation and execution time is on the order of a one group model. It represents a higher order approximation than FLARE, since it is effectively a two-group calculation, and does not require extensive normalization. The applicability of this type of model to BWR analysis has been demonstrated by Børresen<sup>4</sup>.

BWR operation is characterized by the concept of an operating cycle. An operating cycle is a period of operation at the end of which the reactor is shut down for refueling. Approximately one quarter of the most highly depleted fuel assemblies are replaced at this time in order to provide a sufficient amount of reactivity to sustain operation through the next cycle. Some important decisions that must be made by those concerned with fuel management are the length of the cycle, enrichment of the reload fuel, the number of assemblies to be replaced, and the rearrangement of assemblies within the core. A reactor simulator can be used to evaluate the consequences of these types of fuel management decisions.

The primary limiting factor which must be dealt with in reactor core design is the critical heat flux. This is a limiting heat flux

beyond which a sharp jump in the surface temperature of the fuel cladding occurs. The ratio of the critical heat flux to the maximum local heat flux is called the minimum critical heat flux ratio (MCHFR). A MCHFR calculation, using a design limit correlation, has been included in the code. Recent BWR designs have used a MCHFR design limit of 1.9. The reactors are designed so that critical heat flux ratios will not fall below this limit during normal operation.

Another important consideration in reactor design and operation is the power peaking ratio. It is defined as the ratio of the peak to average local power densities in a reactor. This is important for several reasons. Reactor fuel and thermal-hydraulic performance are generally improved by reducing power peaking ratios. Operation with a large power peak accelerates fuel depletion in the region of the peak. Excessive local fuel exposures incur an increased risk of fuel failure. Power peaking also limits the core power density, since design limits, such as the MCHFR limit, must not be exceeded at the power peak. Typical power peaking ratio design limits are 1.5 in the axial direction, 1.4 radially, and 2.6 overall.

BWR power peaking occurs in regions of relatively high reactivity, such as in freshly loaded or highly enriched fuel, and near the bottom of a core due to voiding. It can be controlled by inlet orificing, burnable poison distributions, and by the choice of fuel loading pattern and control rod positions.

The economic benefits of a reduction in power peaking can be very large. This is particularly true of nuclear generating systems built using the stretch concept<sup>5</sup>. Such a system is built with a turbine gen-

erator capable of handling a boiler rated at a higher output than the nominal output of the reactor. The purpose of this was to allow for expected improvements in technology and operating limits. A reduction in power peaking permits operation at a higher power density and increases the capacity of the generating system. Higher than expected power peaking could also force operation at less than the nominal or designed power density in order to stay within safe operating limits. Fuel fabrication costs can also be reduced if power peaking ratios are low.

A major problem in BWR operation is determining control rod withdrawal patterns which will optimize reactor performance. Optimal performance generally means maintaining the lowest possible power peaking ratio throughout the cycle. Control rods can be used to shape the power distribution, but are ideally fully withdrawn at the end of a cycle.

An option has been included in the code which will calculate the optimal power distribution for an operating cycle. This is called the Haling solution<sup>6</sup>. A Haling solution gives the end of life power and fuel exposure distributions which would result if the same power distribution were maintained throughout the cycle. This is determined from the beginning of cycle fuel loading and exposure distributions and the cycle length in terms of the average fuel exposure increment. If maintained, the Haling power distribution will minimize power peaking throughout the cycle. Haling solutions are useful for comparing fuel loading patterns as well as for determining optimal burnable poison distributions and control rod patterns.

The remainder of this thesis discusses the methods used in develop-

ing the code and results that have been obtained. The neutronics model is described in Chapter 2. Cross section input generation and evaluation are discussed in Chapter 3. Chapter 4 describes the thermal-hydraulic model and Chapter 5 describes the Haling solution logic. Results obtained from benchmarking and testing the code are given in Chapter 6. Conclusions are given in Chapter 7. Input instructions and a source listing are provided in the appendices.

## 2.0 NEUTRONICS MODEL

The reactor power distribution is calculated using a model which represents the reactor as a group of geometrically identical nodes. A node is defined as a homogeneous volume whose boundaries are defined by mesh lines. Mesh lines are chosen to lie between fuel assemblies and along the core periphery. The number of axial nodes represented is optional, but should be chosen such that the node height is approximately equal to the width.

The power distribution calculation is based on a finite difference approximation to the two-group neutron diffusion equations. The fast and thermal fluxes are calculated at points taken to be at the center of each node. An averaging scheme is then used to calculate node averaged fast and thermal fluxes. The power distribution is calculated from the node averaged flux distributions.

### 2.1.1 Formulation of Difference Equations

The two-group neutron diffusion equations may be written as

$$-D_1 \nabla^2 \phi_1 + (\Sigma_{a1} + \Sigma_r) \phi_1 = \frac{1}{k_{\text{eff}}} (\nu \Sigma_{f1} \phi_1 + \nu \Sigma_{f2} \phi_2) \quad (2-1a)$$

$$-D_2 \nabla^2 \phi_2 + \Sigma_{a2} \phi_2 = \Sigma_r \phi_1 \quad (2-1b)$$

$\phi_1, \phi_2$  = fast and thermal fluxes

$D_1, D_2$  = fast and thermal diffusion coefficients

$\Sigma_{a1}, \Sigma_{a2}$  = fast and thermal macroscopic absorption cross sections

$$\begin{aligned}
\Sigma_r &= \text{fast removal cross section} \\
\nu\Sigma_{f1}, \nu\Sigma_{f2} &= \text{fast and thermal } \nu\text{-fission cross sections} \\
k_{\text{eff}} &= \text{effective multiplication factor}
\end{aligned}$$

These coupled equations are not suitable for direct formulation of coarse mesh difference equations. A mesh size characteristic of a BWR fuel assembly pitch ( $\sim 15$  cm) would lead to a break down of the difference approximation for the thermal group. This is due to the comparatively small (3-7 cm) mean free path of thermal neutrons. This is not a problem for the fast group since the mean free path for this group is nearly the same as the assembly pitch.

A zero thermal buckling approximation can be made which will alleviate this problem. This is a reasonably good approximation for large node sizes. Equation 2-1b becomes

$$\phi_2 = \frac{\Sigma_r}{\Sigma_{a2}} \phi_1 \quad . \quad (2-2)$$

Equation 2-2 gives the asymptotic thermal flux solution.

It is now convenient to define a new  $\nu$ -fission cross section.

$$\nu\Sigma_f = \nu\Sigma_{f1} + \frac{\Sigma_r}{\Sigma_{a2}} \nu\Sigma_{f2} \quad (2-3)$$

Using equations 2-2 and 2-3, equation 2-1a can be reformulated as

$$-D_1 \nabla^2 \phi_1 + (\Sigma_{a1} + \Sigma_r) \phi_1 = \frac{1}{k_{\text{eff}}} \nu\Sigma_f \phi_1 \quad . \quad (2-4)$$

The difference equations are obtained by integrating equation 2-4 over the node volume. A mesh point is taken to be at the center of a node of height  $DZ$  and width and depth  $DX$  as shown in figure 2.1.

Integration of the leakage term is accomplished by first trans-

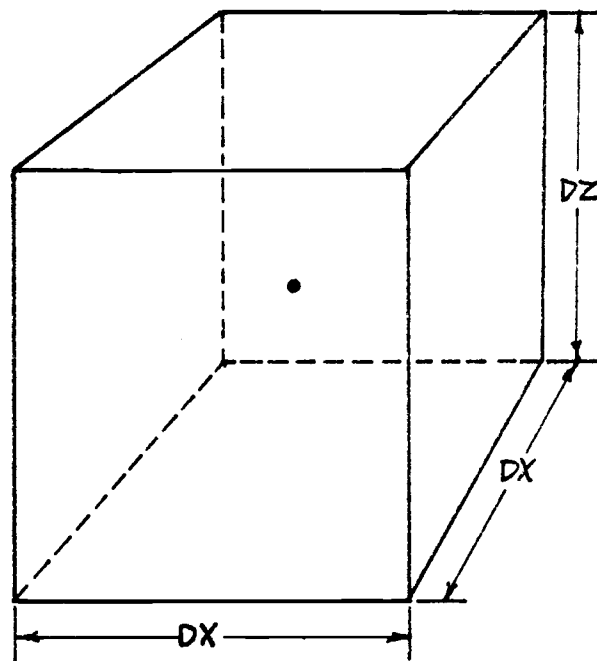


Figure 2.1 Node Dimensions.



forming the volume integral to a surface integral by means of Green's theorem.

$$\int_V \nabla^2 \phi dV = \int_S \nabla \phi dA \quad (2-5)$$

The surface integral is carried out over the six faces of the node.

Evaluation of the flux gradient at the interface is easily understood by considering a one-dimensional case where two mesh points,  $\ell$  and  $\ell+1$ , are separated by a distance,  $d$ , and equidistant from a mesh line separating two regions. The region to the left of the mesh line has diffusion coefficient  $D_\ell$ , and that to the right has  $D_{\ell+1}$ .

Using difference approximations, continuity of the neutron current at the boundary gives

$$\frac{-D_\ell (\phi_{\ell+\frac{1}{2}} - \phi_\ell)}{d/2} = \frac{-D_{\ell+1} (\phi_{\ell+1} - \phi_{\ell+\frac{1}{2}})}{d/2} \quad (2-6)$$

$\phi_\ell$ ,  $\phi_{\ell+\frac{1}{2}}$ , and  $\phi_{\ell+1}$  refer to the fluxes at point  $\ell$ , the mesh line, and point  $\ell+1$  respectively. The flux at the interface is given by

$$\phi_{\ell+\frac{1}{2}} = \frac{D_\ell \phi_\ell + D_{\ell+1} \phi_{\ell+1}}{D_\ell + D_{\ell+1}} \quad (2-7)$$

Using this expression, the flux gradient at the interface can be written in terms of  $\phi_\ell$  and  $\phi_{\ell+1}$ .

$$\nabla \phi_{\ell+\frac{1}{2}} = \left\{ \frac{D_{\ell+1}}{D_\ell + D_{\ell+1}} \right\} \left\{ \frac{\phi_{\ell+1} - \phi_\ell}{d/2} \right\} \quad (2-8)$$

The difference approximation of equation 2-4 for mesh point  $o$  becomes

$$\sum_{i=1}^6 \frac{D_o D_i}{D_o + D_i} \frac{(\phi_o - \phi_i) A_i}{d_i/2} + (\Sigma_{a1} + \Sigma_r)_o \phi_o V_o = \frac{1}{k_{eff}} \nu \Sigma_{fo} \phi_o V_o \quad (2-9)$$

$A_i$  = area of the interface between nodes o and i

$V_o$  = volume of node o

$d_i$  = distance between mesh points o and i

The summation is over all six nodes adjacent to node o as shown in figure 2.2. Since all nodes in the problem have equal dimensions, equation 2-9 can be simplified by dividing by  $V_o$ . It can then be expressed in a more convenient form as

$$\phi_o = \frac{S_o + \sum_{i=1}^6 C_i \phi_i}{C_7} \quad (2-10)$$

$$S_o = \frac{1}{k_{eff}} v \Sigma_{fo} \phi_o \quad (2-11)$$

$$C_i = \frac{2D_o D_i}{d_i^2 (D_o + D_i)} \quad (2-12)$$

$$C_7 = (\Sigma_{a1} + \Sigma_r)_o + \sum_{i=1}^6 C_i \quad (2-13)$$

### 2.1.2 Boundary Conditions

Boundary conditions are treated by specifying the flux at a node's boundary interface as a fraction of the flux at the mesh point of that node. Consider a node with mesh point  $\ell$  whose interface lies on the problem boundary. An imaginary node with mesh point  $\ell+1$ , and with consistent mesh spacing is assumed to lie beyond the boundary as shown in figure 2.3. The nuclear properties of node  $\ell+1$  are assumed identical to those of node  $\ell$ . From equation 2-7, the flux at the boundary is simply

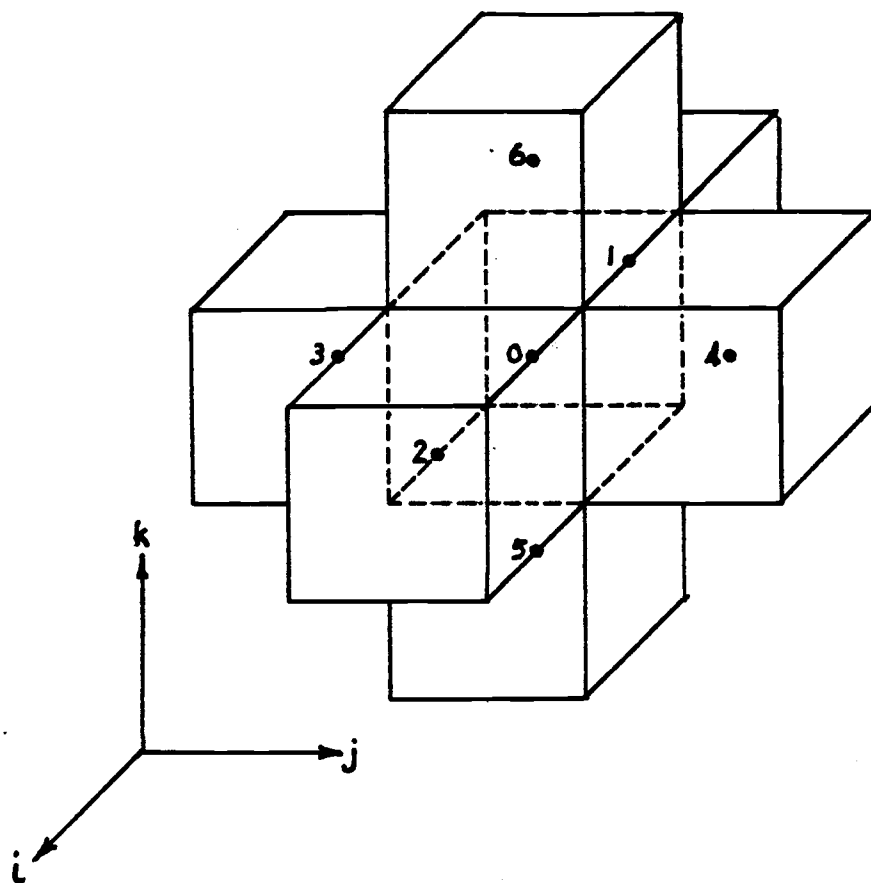


Figure 2.2 Schematic of Three-Dimensional Node Geometry.

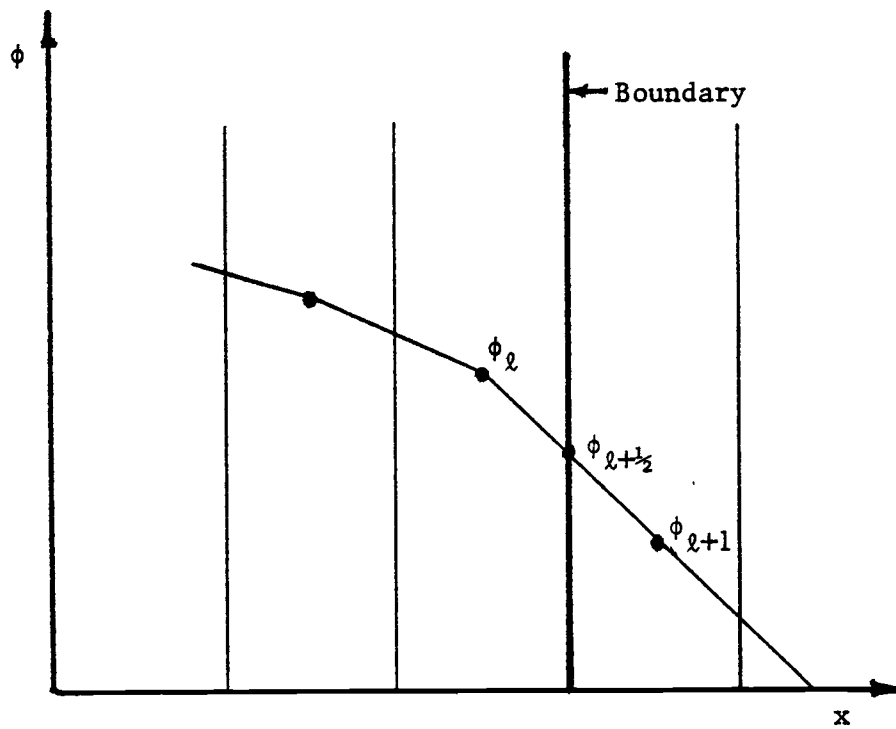


Figure 2.3 Boundary Condition Representation.

$$\phi_{\ell+1/2} = \frac{1}{2}(\phi_{\ell} + \phi_{\ell+1}) \quad . \quad (2-14)$$

Specifying the boundary flux as

$$\phi_{\ell+1/2} = ALB \cdot \phi_{\ell} \quad (2-15)$$

gives

$$\phi_{\ell+1} = (2ALB - 1)\phi_{\ell} \quad . \quad (2-16)$$

With prior knowledge of  $\phi_{\ell+1}$  in the form of equation 2-16, an extra term,  $(2/d_i^2)D_i(1 - ALB)$ , can be added to the denominator of equation 2-10. One such term is added for each node interface that lies on the problem boundary. The summations in equations 2-10 and 2-13 then correspond to the remaining node interfaces which do not lie on a boundary.

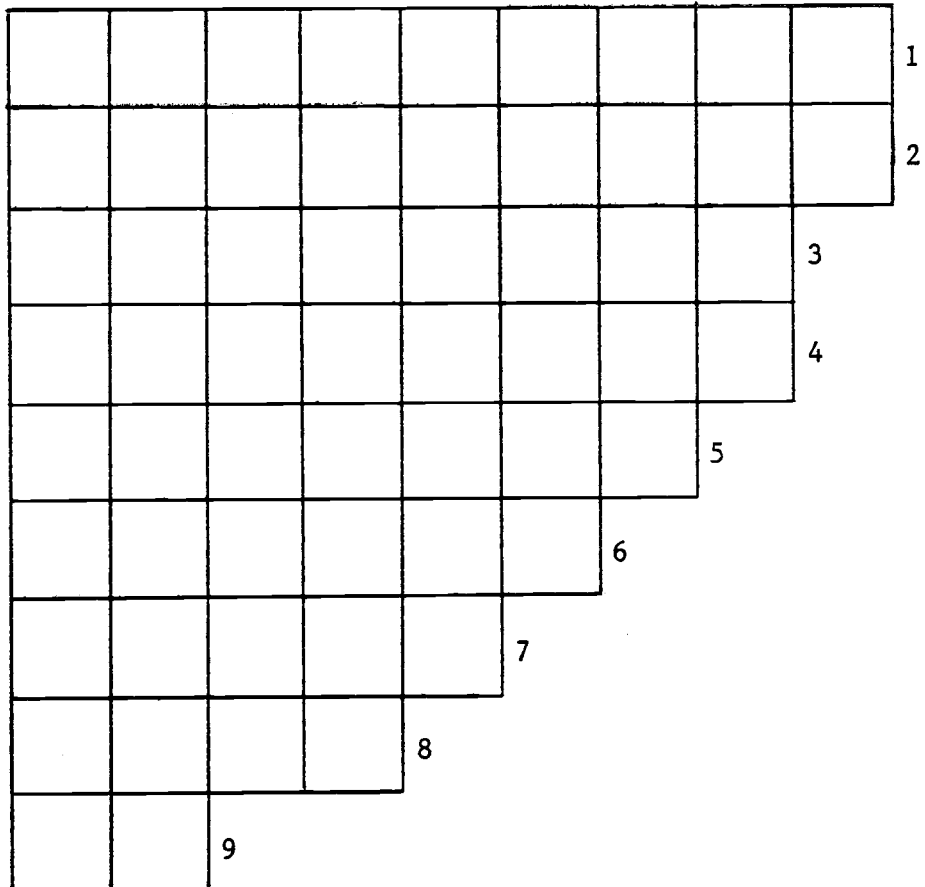
From equation 2-15 it is obvious that  $ALB = 1.0$  corresponds to a zero current, or reflecting, boundary condition and  $ALB = 0.0$  corresponds to a zero flux boundary condition. Thus, as  $ALB$  implies, it can be conveniently thought of as an albedo (although it is not in a strict sense of the word).

Boundary conditions are required for the core bottom and top and each of the radial faces of the core periphery. A problem represented by  $NX$  nodes in the radial direction requires  $NX$  separate boundary conditions to represent the complex geometry of the periphery. The  $NX$  boundaries are indexed as shown in figure 2.4.

In order to account for the increased leakage caused by voiding, boundary conditions are input as a function of void fraction.

$$ALB = A + B \cdot VOID(\ell) \quad (2-17)$$

$A$  and  $B$  are input constants for each boundary.  $VOID(\ell)$  is the void fraction of the appropriate boundary node. Separate boundary conditions



Number of radial nodes,  $NX = 9$ .

Figure 2.4 Boundary Node Indexing Convention.

are input for the fast and thermal fluxes.

### 2.1.3 Solution

Equation 2-4 is solved using the inner-outer iteration technique illustrated in figure 2.5. Using an initial flux distribution guess, a fission source distribution is calculated. Holding the source distribution fixed, MAXI inner iterations are carried out using equation 2-10. (MAXI is an input constant.) The eigenvalue for the next outer (source) iteration is calculated from

$$k_{\text{eff}}^N = k_{\text{eff}}^{N-1} \frac{TF^N}{TF^{N-1}} \quad . \quad (2-17)$$

$TF^N$  = total fission source for iteration N

N = outer iteration counter

A new source distribution is then calculated as in equation 2-11 and the inner iteration is repeated. The outer iteration is terminated when the source distribution is converged or when the number of outer iterations is equal to an input limit, MAXO. The outer iteration is considered converged when the maximum relative change in nodal fission sources is less than or equal to an input convergence criterion, EPSO.

Convergence of the source iteration is accelerated by successive over-relaxation (SOR). The accelerated source distribution is calculated from

$$S_{\ell}^{N*} = S_{\ell}^{N-1} + \alpha(S_{\ell}^N - S_{\ell}^{N-1}) \quad . \quad (2-18)$$

$S^{N*}$  = accelerated source for outer iteration N

$\alpha$  = over-relaxation factor

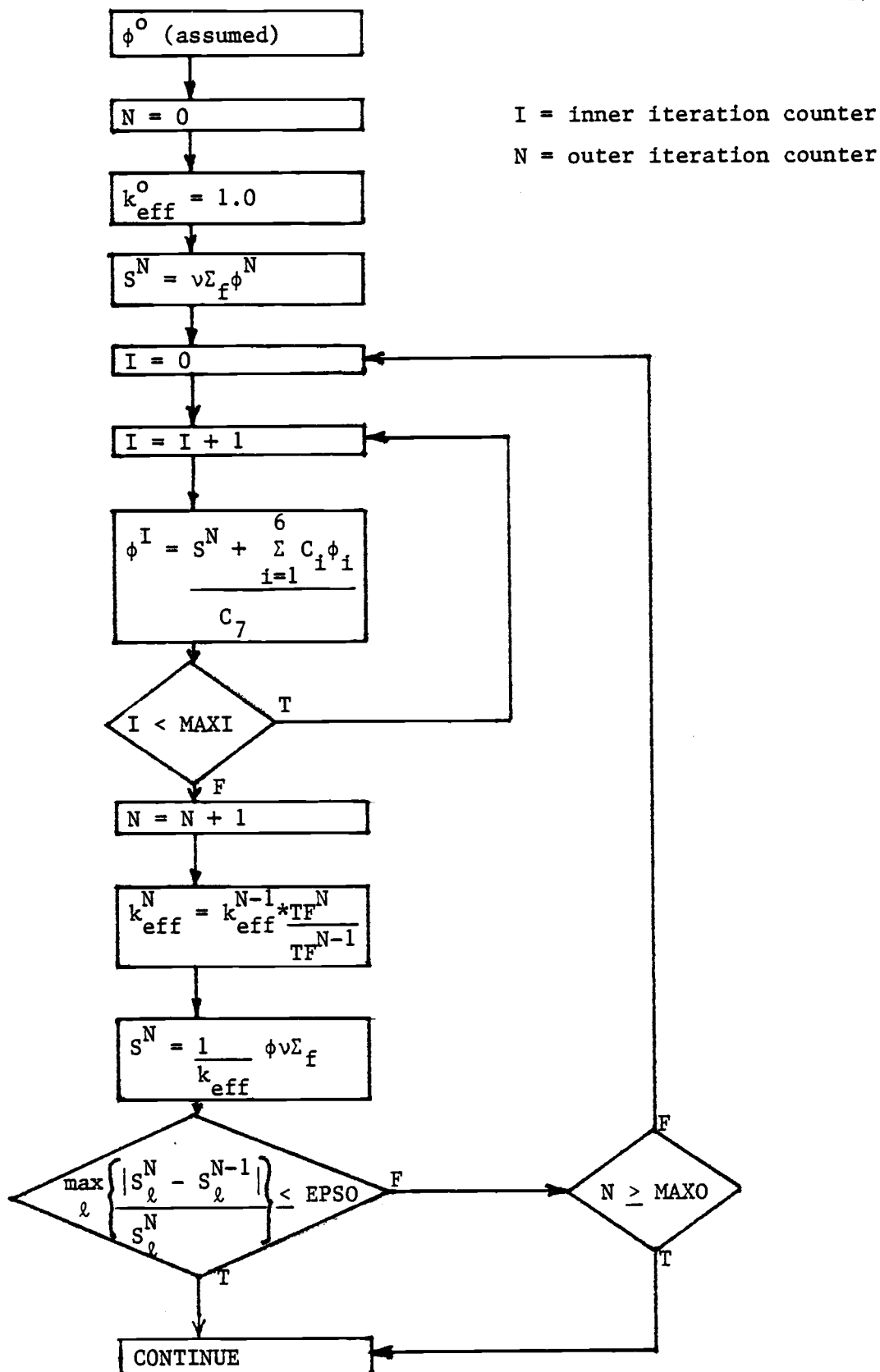


Figure 2.5 Flow Diagram of Inner and Outer Iterations.



The over-relaxation factor is input. A value of 1.65 is close to optimum for most cases and is the default value.

## 2.2 Node Averaging

A coarse mesh diffusion theory model tends to under-predict current densities at node interfaces. This is particularly true if there is a large difference between the diffusion coefficients of the two adjacent nodes or if the node is on the core periphery. This can lead to a fairly large error in the predicted power distribution. In order to reduce this error, a node averaging scheme proposed by Børresen<sup>4</sup> is employed.

The node averaged flux,  $\bar{\phi}$ , is calculated as a weighted average of the midpoint flux and the fluxes at the node interfaces.

$$\bar{\phi}_O = b \cdot \phi_O + c \left( \sum_{i=1}^4 \psi_i + R \cdot \sum_{k=1}^2 \psi_k \right) \quad (2-19)$$

$$b = \frac{3a}{3a + (1-a)(R+2)} \quad (2-20)$$

$$c = \frac{1-a}{6a + 2(1-a)(R+2)} \quad (2-21)$$

$$R = \frac{DX^2}{DZ^2} \quad (2-22)$$

$\phi_O$  = flux at node midpoint

$\psi$  = flux at node interface

$a$  = input weighting factor

$DX$  = horizontal mesh width

$DZ$  = vertical mesh width

The first summation in equation 2-19 corresponds to interfaces with the

four horizontally adjacent nodes. The second corresponds to interfaces with the two vertically adjacent nodes.

For interfaces not on the problem boundary, interface fluxes are calculated as in equation 2-7. For the interface between node o and node i

$$\psi_i = \frac{D_o \phi_o + D_i \phi_i}{D_o + D_i} \quad . \quad (2-23)$$

For interfaces lying on the problem boundary, the interface flux is calculated as in equation 2-15.

$$\psi_i = ALB_i \cdot \phi_o \quad (2-24)$$

$ALB_i$  is the appropriate boundary condition for that face.

Two different values of the weighting factor,  $a$ , are used to calculate the node averaged fast and thermal fluxes. Børresen recommends 0.3 and 0.7 for the fast and thermal groups respectively.

The procedure used in calculating node averaged fluxes is to first calculate node averaged fast fluxes from the fast flux distribution produced by the diffusion theory model. An asymptotic thermal flux distribution is calculated from the node averaged fast flux distribution. Then a node averaged thermal flux distribution is calculated from these asymptotic thermal fluxes.

### 2.3 Power Distribution and Burnup Calculations

A relative power distribution is calculated from the node averaged fast and thermal fluxes. The relative power at node o is given by

$$P_o = \frac{(\kappa \Sigma_{f1} \phi_1 + \kappa \Sigma_{f2} \phi_2)_0}{N} \quad . \quad (2-24)$$

$$\frac{1}{N} \sum_{\ell=1} (\kappa \Sigma_{f1} \phi_1 + \kappa \Sigma_{f2} \phi_2)_{\ell}$$

$N$  = number of nodes represented in the problem

The cross sections used to calculate a flux and power distribution are themselves functions of power due to the feedback effects of void fraction and xenon concentration. The power distribution is solved iteratively. After a power distribution has been calculated, new cross sections consistent with this distribution are calculated. Another power distribution is calculated with the new cross sections, and so on, until convergence is achieved.

In a burnup step calculation, a power distribution is calculated consistent with an initial exposure distribution. Assuming the power distribution remains constant during the input exposure interval, the exposure distribution at the end of the step can be calculated. The exposure at each node is calculated from

$$E_{\ell} = E_{\ell}^i + P_{\ell} \cdot \Delta E \quad . \quad (2-25)$$

$E_{\ell}$  = fuel exposure at end of step (MWD/MTU)

$E_{\ell}^i$  = exposure at beginning of step

$P_{\ell}$  = relative power of node  $\ell$

$\Delta E$  = length of core burnup step (MWD/MTU)

The entire power distribution and burnup time-step solution procedure is illustrated in figure 2.6. Starting with the initial fuel exposure distribution and guesses of the power and void distributions, the code calculates nodal cross sections and then the corresponding flux and power distributions. A new void distribution is calculated

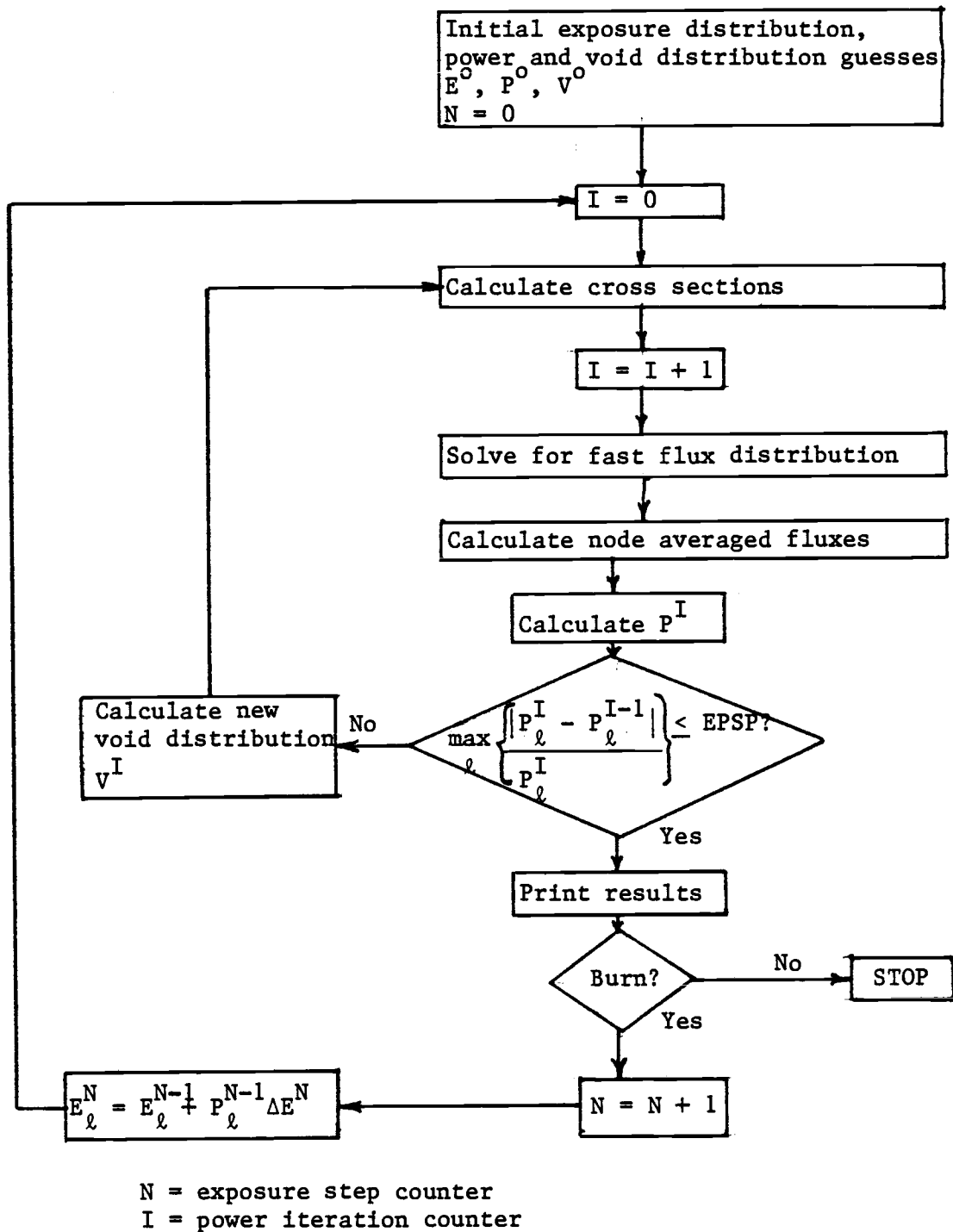


Figure 2.6 Flow Diagram of Power Iteration and Exposure Step Calculation

and the iteration continues until the maximum relative change in nodal power is less than an input convergence criterion, EPSP. The results are printed and if another burnup time-step is requested, the exposure array is updated before repeating the process.

The void-power iteration is sensitive to large perturbations in successive power distribution iterants. Requiring tight convergence on the source iteration when the power distribution is not yet close to being converged can cause an instability and divergence in the power iteration. The instability can be avoided by setting a low limit on the maximum number of source iterations per power iteration (MAX0). This minimizes perturbations between early iterants. If the code detects divergence in the power iteration, it will automatically decrease the limit on the maximum number of source iterations until convergence resumes.

### 3.0 CROSS SECTIONS

This chapter discusses the generation of two-group cross section input and the manner in which the code evaluates appropriate cross sections for each node.

#### 3.1 Generation of Two-Group Input

The two-group constants required as input must be able to yield the appropriate node averaged reaction rates when multiplied by node averaged fast and thermal fluxes. This requires that the flux spectrum within each energy group be accounted for as well as the detailed flux variation due to heterogeneities within the node volume. This is done by flux averaging.

The node averaged flux for energy group  $i$  is defined as

$$\bar{\phi}_i = \left( \int_V d^3r \int_{u_{i-1}}^{u_i} \partial u \phi(r, u) \right) \div \left( \int_V d^3r \right) \quad (3-1)$$

$\phi(r, u)$  is the lethargy dependent flux at point  $r$ . The volume integral is carried out over the node volume and the lethargy integral from the lower to the upper lethargy bounds defining group  $i$ . In a two-group model, the fast group commonly corresponds to the energy range from 10 Mev to 0.625 ev while the thermal group represents energies below 0.625 ev.

A cross section consistent with equation 3-1 would be evaluated as

$$\Sigma_i = \frac{\int_V d^3r \int_{u_{i-1}}^{u_i} \Sigma(r, u) \phi(r, u) \partial u}{\int_V d^3r \int_{u_{i-1}}^{u_i} \phi(r, u) \partial u} \quad (3-2)$$

One method for calculating these cross sections will be briefly described here. It uses the LEOPARD<sup>7</sup> and PDQ/HARMONY<sup>8,9</sup> codes. For a more detailed discussion of multigroup constant generation see Duderstadt and Hamilton<sup>10</sup>.

LEOPARD generates exposure dependent few group cross sections for a homogenized pin cell. A pin cell is a two-dimensional representation of a single fuel rod and the surrounding moderator region as it appears in the array of a fuel assembly. While the LEOPARD calculation is zero-dimensional and primarily spectral, the flux depression in the fuel rod is accounted for by means of disadvantage factors.

PDQ is a few-group diffusion theory code which uses the HARMONY depletion system. Using the few group output from LEOPARD, it can be used to model a fuel assembly in two dimensions. The spatial flux variation in the assembly can be accounted for by using PDQ to flux weight cross sections over this two-dimensional model. Using its depletion capabilities to account for the changes in isotopic concentrations, it can generate exposure dependent two-group cross sections suitable for a node representing the modeled assembly.

A node volume containing a highly absorbing region, such as a burnable poison curtain, may require special treatment. This is due to the limitations of diffusion theory for such regions. A transport theory calculation can be used in such a case.

### 3.2 Table Assignment

Two-group constants are input as table sets at various void fractions and exposures. One table set is required for each unique node

type represented. Several table sets may be required to model one assembly. This is due to the axial burnable poison distributions used in most BWR's.

The following definitions will be used in this thesis. Each unique fuel assembly type is referred to as a fuel type. Fuel types are assigned to various locations in the core model using a two-dimensional array called a fuel type overlay. Cross section table sets are assigned to each axial node in a given fuel type using a table set to fuel type overlay.

Using the table sets assigned to each node, the code is able to calculate the appropriate two-group constants as a function of void fraction, fuel exposure, and equilibrium xenon concentration. These calculations will be described in detail in the following sections.

### 3.3 Evaluation of Nodal Two-Group Constants

Cross sections for a particular node are calculated as follows. Cross sections as a function of void fraction and exposure are calculated from the appropriate table set. This accounts for changes in the neutron energy spectrum and in isotopic concentrations caused by coolant voiding and fuel depletion. Corrections are then made to these cross sections to account for the presence of a control rod, if the node is rodded, and to correct for the amount of equilibrium xenon present.

#### 3.3.1 Void and Exposure Dependency

Cross sections as a function of average node void fraction and exposure are calculated from the input table sets by linear interpolation



or extrapolation. The set of exposure points used in the table sets is referred to as an exposure mask. This must be chosen such that linear interpolation between any two consecutive exposure points will yield acceptably accurate results over the range of fuel exposures likely to be encountered in a problem. Relatively fine exposure steps must be used at low exposures due to the fairly rapid changes in isotopic concentrations occurring in this range.

At each exposure point, cross sections are input at a number of void fractions referred to as a void mask. An acceptable void mask requires a minimum of three (preferably four or five) void fractions equally spaced over the void fraction range likely to be encountered.

### 3.3.2 Control Rod Model

BWR control rods are cruciform shaped, with blades situated between four adjacent fuel assemblies (see figure 3.1). Rods are inserted from the bottom of the core. In order to model the effect of a control rod inserted a distance of N axial mesh spaces from the core bottom, the two-group constants of the first N axial nodes of the four adjacent fuel assemblies must be modified to reflect the change in nuclear properties.

The effect of a control rod on the nuclear properties of a node can be expressed in terms of control rod worth. This is defined as

$$\Delta\rho = \frac{k_{\infty}^{UR} - k_{\infty}^R}{k_{\infty}^{UR}} \quad . \quad (3-3)$$

$\Delta\rho$  = control rod worth

$k_{\infty}^{UR}$  = infinite multiplication factor for unrodded node

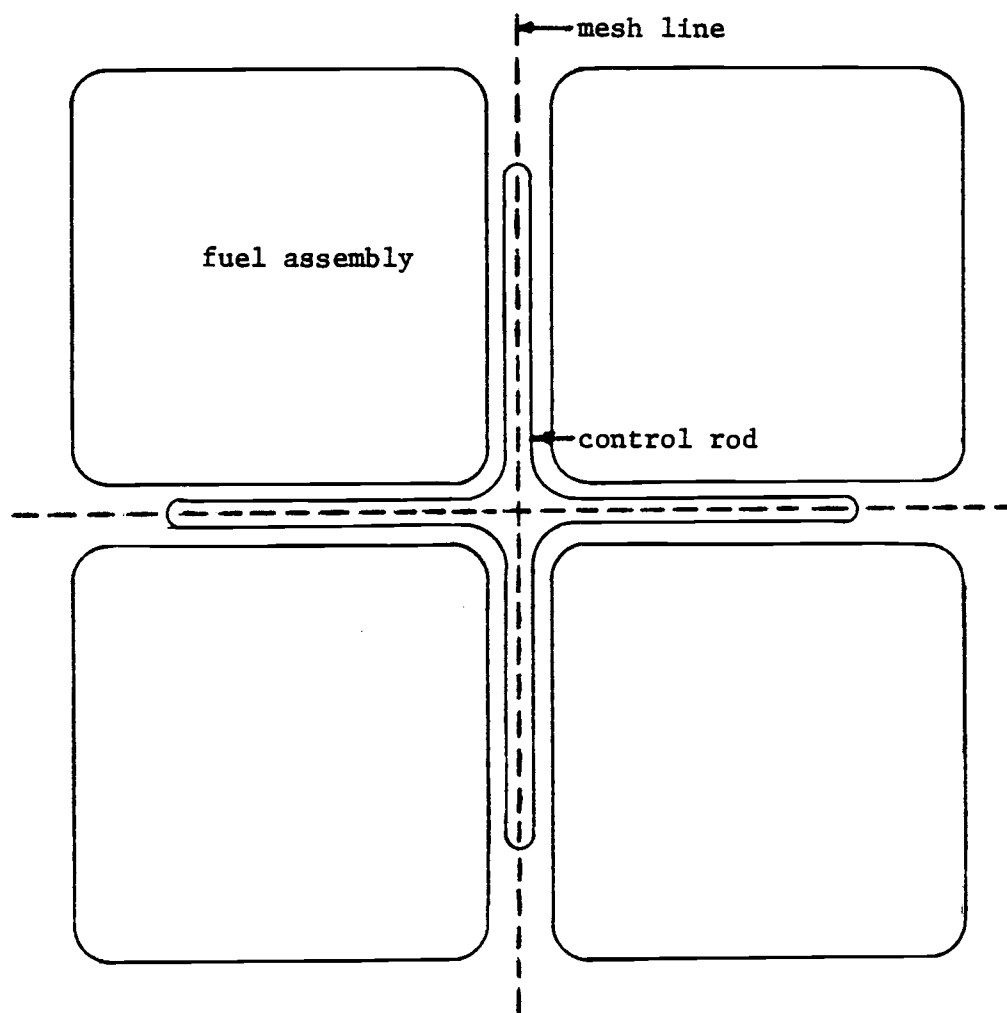


Figure 3.1 BWR Control Rod Geometry.

$k_{\infty}^R$  = infinite multiplication factor for rodded node

$k_{\infty}$  can be expressed in terms of two-group constants as

$$k_{\infty} = \frac{\frac{v\Sigma_{f1} + \Sigma_r v\Sigma_{f2}}{\Sigma_{a2}}}{\Sigma_{a1} + \Sigma_r} \quad (3-4)$$

Control rod worth will increase as the void fraction of a node increases and will also vary as a function of exposure.

The major effect of a control rod is a large increase in the thermal absorbtion of a node. Other neutronic properties are effected to a much lesser extent. The technique employed for modeling a rodded node is to absorb the control rod worth entirely into the thermal absorbtion cross section. All other two-group constants remain the same as for the unrodded case. Thus, from equations 3-3 and 3-4,

$$\Sigma_{a2}^R = \frac{\Sigma_r v\Sigma_{f2}}{(1-\Delta\rho) \left[ \frac{v\Sigma_{f1} + \Sigma_r v\Sigma_{f2}}{\Sigma_{a2}} \right] - v\Sigma_{f1}} \quad (3-5)$$

$\Sigma_{a2}^R$  = thermal absorbtion cross section of rodded node

All constants on the right hand side of equation 3-5, except  $\Delta\rho$ , are those of the unrodded node.

Control rod worths are input for each cross section table set as a table of rod worth versus void and exposure using the same exposure and void points used in the table sets. The worth at a given exposure and void fraction is calculated by linear interpolation. The rod position for each fuel assembly location is input as an integer, greater than or equal to zero, indicating the rod position in terms of the number of

nodes from the core bottom.

### 3.3.3 Equilibrium Xenon Correction

A correction is made to account for the equilibrium Xe-135 concentration in a node for two reasons. First of all, Xe-135 has an extremely large thermal absorption cross section, typically about 1.6 million barns. Secondly, the equilibrium xenon concentration in a node is a function of the neutron flux in that node. Variations in reactor power as well as variations in power density from node to node require that the Xe-135 contribution to the thermal absorption cross sections, inherent in the table sets, be corrected. The effect on all other cross sections is negligibly small.

The Xe-135 corrected thermal absorption cross section is calculated at each node from

$$\Sigma_{a2} = \Sigma_{a2}^{\text{No Xe}} + N^{\text{Xe}} \sigma_{a2}^{\text{Xe}} \quad (3-6)$$

$\Sigma_{a2}^{\text{No Xe}}$  = thermal absorption cross section with the Xe-135 contribution removed

$N^{\text{Xe}}$  = number density of Xe-135 ( $\text{cm}^{-3}$ )

$\sigma_{a2}^{\text{Xe}}$  = Xe-135 microscopic thermal absorption cross section ( $\text{cm}^2$ )

Xe-135 number densities and microscopic thermal absorption cross sections are included in the input interpolating table sets.  $\Sigma_{a2}^{\text{No Xe}}$  is calculated at each of the interpolating exposure and void fraction state points by subtracting  $N^{\text{Xe}} \sigma_{a2}^{\text{Xe}}$  from the corresponding macroscopic thermal absorption cross section.

In terms of two-group constants, the number density of Xe-135 is governed by the following pair of differential equations.

$$\frac{dN^I}{dt} = \gamma_I(\phi_1 \Sigma_{f1} + \phi_2 \Sigma_{f2}) - N^I \lambda_I \quad (3-7)$$

$$\begin{aligned} \frac{dN^{Xe}}{dt} = & \gamma_{Xe}(\phi_1 \Sigma_{f1} + \phi_2 \Sigma_{f2}) + N^I \lambda_I \\ & - N^{Xe}(\phi_1 \sigma_{a1}^{Xe} + \phi_2 \sigma_{a2}^{Xe}) - N^{Xe} \lambda_{Xe} \end{aligned} \quad (3-8)$$

$N^{Xe}, N^I$  = Xe-135 and I-135 number densities

$\gamma_{Xe}, \gamma_I$  = Xe-135 and I-135 fission yields

$\lambda_{Xe}, \lambda_I$  = Xe-135 and I-135 decay constants

$\sigma_{a1}^{Xe}, \sigma_{a2}^{Xe}$  = fast and thermal microscopic absorption cross sections for Xe-135

The equilibrium Xe-135 number density equation used by the code is obtained by setting the time derivatives in equations 3-7 and 3-8 to zero. The Xe-135 fast absorption cross section is neglected since it is relatively small (8 barns). The xenon transients which follow a change in power density are ignored in this model. These transients are on the order of eight hours and are not important in most burnup calculations. For each node, the Xe-135 number density is calculated from

$$N^{Xe} = \frac{\gamma_{I+Xe}(\phi_1 \Sigma_{f1} + \phi_2 \Sigma_{f2})}{\lambda_{Xe} + \phi_2 \sigma_{a2}^{Xe}} \quad (3-9)$$

$$\lambda_{Xe} = 2.0997 \times 10^{-5} \text{ sec}^{-1}$$

The I-135 + Xe-135 fission yield is input as a table versus exposure for each table set. The appropriate yield is calculated by interpolation.  $\Sigma_{f1}$ ,  $\Sigma_{f2}$ , and  $\sigma_{a2}^{Xe}$  are calculated by interpolation versus void

fraction and exposure.

The fast and thermal fluxes from the diffusion theory calculation are only relative values and must be converted to absolute fluxes before they can be used in equation 3-9. The absolute fluxes,  $\phi_1^a$  and  $\phi_2^a$ , are calculated from

$$\phi_1^a = CF \cdot \phi_1 \quad , \quad (3-10)$$

$$\phi_2^a = CF \cdot \phi_2 \quad . \quad (3-11)$$

The conversion factor, CF, is calculated from

$$CF = \frac{P_{th} \cdot 6.24146 \times 10^{18} \frac{\text{Mev/sec}}{\text{Mw}}}{N \cdot \text{Vol} \cdot \sum_{\ell=1} (\phi_1 \kappa \Sigma_{f1} + \phi_2 \kappa \Sigma_{f2})_{\ell}} \quad . \quad (3-12)$$

$P_{th}$  = power generated by the portion of the core being modeled  
( $Mw_{th}$ )

$\text{Vol}$  = node volume ( $\text{cm}^3$ )

$\kappa \Sigma_f$  =  $\kappa$ -fission cross section (Mev/cm)

The summation in the denominator of equation 3-12 is over all nodes in the problem.

#### 4.0 THERMAL-HYDRAULIC TREATMENT

A typical BWR coolant flow diagram is shown in figure 4.1. Sub-cooled water enters the core from the bottom. Inlet orifices direct the flow to the fuel channels and serve to restrict the flow in low power assemblies, primarily those on the core periphery. The "active" coolant flows up through the fuel channels. The small amount of coolant that flows between the fuel channels is termed bypass flow.

As coolant flows up a fuel channel, it is heated to saturation and becomes a two-phase mixture. The flow quality,  $X$ , at a given point in the channel is defined as the mass flow rate of the vapor phase divided by the mass flow rate of the mixture.

The void fraction is defined as the ratio of the local vapor volume to the total local volume. At a given point in a flow channel it can be expressed as

$$\alpha = \frac{A_v}{A_v + A_l} \quad . \quad (4-1)$$

$\alpha$  = void fraction

$A_v$  = cross sectional area occupied by vapor phase

$A_l$  = cross sectional area occupied by liquid phase

The density of a two-phase mixture can be related to the void fraction by

$$R = 1. + \alpha(\rho_v/\rho_l - 1.) \quad . \quad (4-2)$$

$\rho_v$  = density of saturated vapor

$\rho_l$  = density of saturated liquid

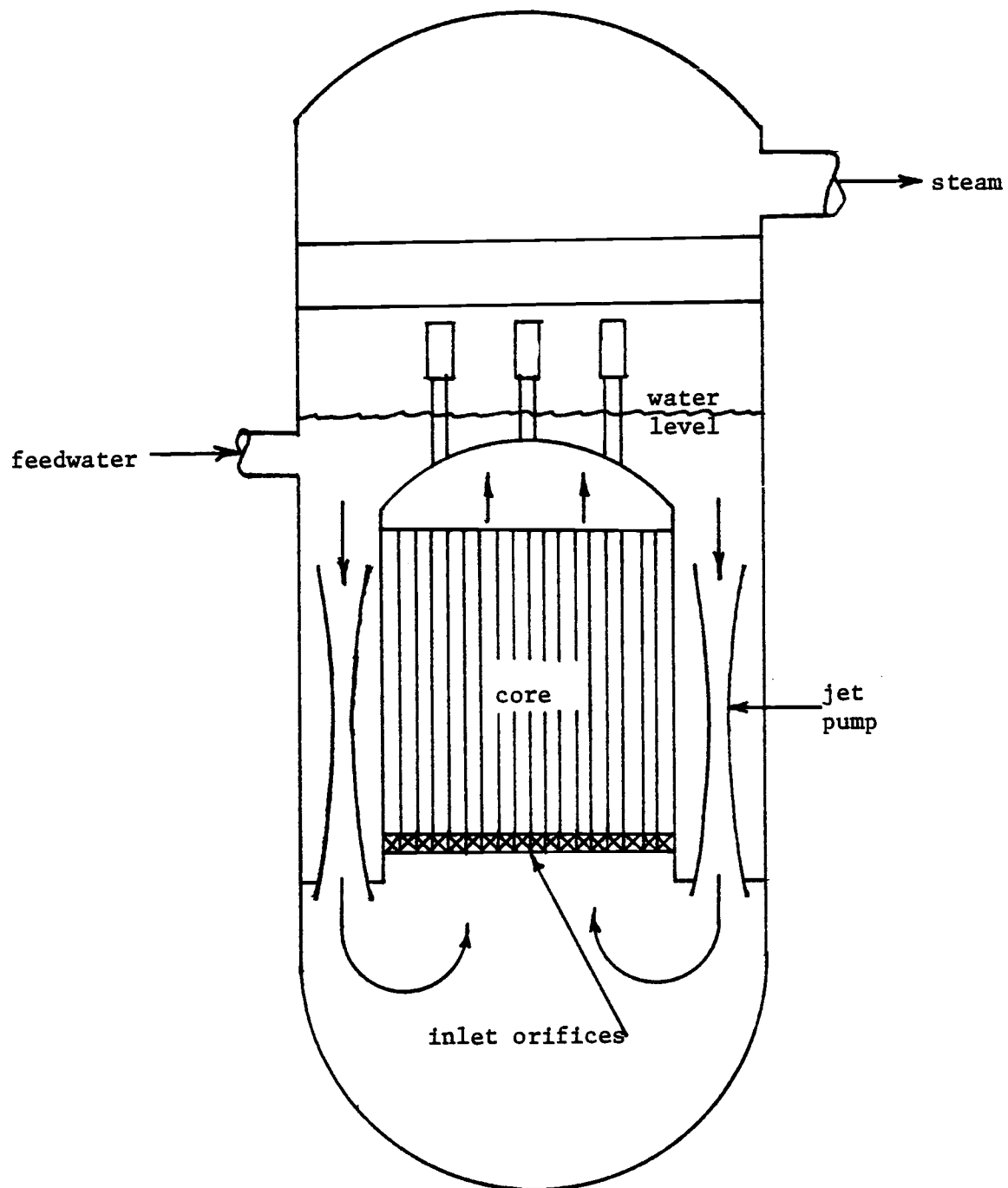


Figure 4.1 BWR Coolant Flow Diagram.



$R$  = density of two-phase mixture relative to  $\rho_\ell$

As mentioned previously, a thermal-hydraulic feedback calculation is linked to the neutronics model. This is an iterative process by which new nodal nuclear properties are calculated as a function of the void fraction of the active coolant flow for each power distribution iterant. A minimum critical heat flux ratio (MCHFR) calculation has also been included in the thermal-hydraulics model. The calculational techniques used in the thermal-hydraulics model will be discussed in this chapter.

#### 4.1 Coolant Flow Distribution

In order to calculate the quality of the coolant at a particular node, the coolant mass flow rate within the flow channel must be known. The flow rate varies considerably from channel to channel. This is primarily due to inlet orificing and variations in void content. The void content is, of course, coupled to the power generated in the assembly. In order to account for these effects, a polynomial fit of flow versus average assembly power is used.

$$F_r(I,J) = A + B\bar{P}(I,J) + C\bar{P}(I,J)^2 \quad (4-3)$$

$F_r(I,J)$  = relative coolant mass flow rate in channel (I,J)  
at hot full power conditions

$\bar{P}(I,J)$  = assembly averaged power, relative to an "average"  
assembly at hot full power

A,B,C = input constants

A, B, and C are input for each hydraulically unique flow channel type. These constants can be evaluated using a separate thermal-hydraulic code

capable of treating, in detail, effects such as inlet orifices, fuel rod array geometry, flow mixers, and the axial power distribution. This may require iteration between assembly power distributions and the flow polynomials. A channel type overlay is used to assign a set of flow constants to each channel location.

The assembly averaged power used in equation 4-3 is calculated from

$$\bar{P}(I,J) = \frac{PCTPWR}{100.} \frac{1.}{NZ} \sum_{k=1}^{NZ} P(I,J,k) \quad . \quad (4-4)$$

PCTPWR = reactor power in percent of rated power

NZ = number of axial nodes represented

P(I,J,K) = relative power of node (I,J,K)

After calculating flow rates from equation 4-3, the values are normalized to a mean of 1.0. The actual flow rate is then calculated from

$$F(I,J) = \frac{TFLOW}{Na} F_r(I,J) \quad . \quad (4-5)$$

F(I,J) = actual coolant mass flow rate in lbm/hr

TFLOW = total incore coolant flow rate (corrected for bypass flow) in lbm/hr

Na = number of assemblies in the core

## 4.2 Quality Calculation

The quality of the active coolant in a flow channel is calculated from

$$X = \frac{h - h_{sat}}{h_{fg}} \quad . \quad (4-6)$$

h = coolant enthalpy (Btu/lbm)

$h_{\text{sat}}$  = enthalpy of saturated water

$h_{\text{fg}}$  = heat of vaporization

The increase in coolant enthalpy for a given node,  $\Delta h(I,J,K)$ , can be calculated from

$$\Delta h(I,J,K) = \left[ \frac{\text{PCTPWR} \cdot \text{MWTH} \cdot \text{CF}}{100 \cdot \text{NZ}} \right] \left[ \frac{P(I,J,K)}{F(I,J)} \right] \quad (4-7)$$

PCTPWR = reactor power in percent of rated power

MWTH = rated reactor power ( $Mw_{\text{th}}$ )

CF =  $3.4127 \times 10^6 \frac{\text{Btu/hr}}{Mw}$

NZ = number of axial nodes in problem

$P(I,J,K)$  = relative power of node (I,J,K)

$F(I,J)$  = coolant flow rate through node (lbm/hr)

The coolant enthalpy at the exit of a node is given by

$$h_{\text{exit}}(I,J,K) = h_{\text{in}} + \sum_{k=1}^K \Delta h(I,J,k) \quad (4-8)$$

where  $h_{\text{in}}$  is the core inlet enthalpy. The average enthalpy of a node,  $\bar{h}(I,J,K)$ , is used to calculate the quality of that node.

$$\bar{h}(I,J,K) = \frac{1}{2} \left[ h_{\text{exit}}(I,J,K-1) + h_{\text{exit}}(I,J,K) \right] \quad (4-9)$$

The numerical expression of equation 4-6 is then

$$X(I,J,K) = \frac{1}{h_{\text{fg}}} \left\{ h_{\text{in}} + \left[ \frac{\text{PCTPWR} \cdot \text{MWTH} \cdot \text{CF}}{100 \cdot \text{NZ}} \right] \frac{1}{F(I,J)} \right. \\ \left. \times \left[ \sum_{k=1}^{K-1} P(I,J,k) + \frac{1}{2} P(I,J,K) \right] - h_{\text{sat}} \right\} \quad (4-10)$$

### 4.3 Void Fraction

The void fraction of a node is calculated from the average node quality using the modified Armand correlation<sup>11</sup>.

$$\alpha = 0. \quad \text{for } X \leq 0.$$

$$\alpha = \frac{C_1 \cdot X + C_2 \cdot X^2}{\frac{v_f}{v_g} + X \cdot \left[ 1. - \frac{v_f}{v_g} \right]} \quad \text{for } X > 0. \quad (4-11)$$

$v_f$  = specific volume of saturated liquid

$v_g$  = specific volume of saturated steam

$C_1$  and  $C_2$  are empirical constants. The default values are

$$C_1 = 0.833 \quad ,$$

$$C_2 = 0.167 \quad .$$

A graph of void fraction versus quality using the above values for  $C_1$  and  $C_2$  and specific volumes for saturated steam and water at 1035 psia and 548.8° Fahrenheit is shown in figure 4.2.

### 4.4 MCHFR Calculation

The local critical heat flux is calculated using the Janssen-Levy<sup>12</sup> correlation. At 1000 psia,

$$q''_{\text{crit}} = 7.05 \times 10^5 + 0.237 \cdot G \quad \text{for } X < x_1 \quad (4-12)$$

$$q''_{\text{crit}} = 1.635 \times 10^6 - 0.270 \cdot G - 4.710 \times 10^6 \cdot X \quad \text{for } x_1 < X < x_2$$

$$q''_{\text{crit}} = 6.05 \times 10^5 - 0.164 \cdot G - 6.53 \times 10^5 \cdot X \quad \text{for } x_2 < X$$

$$q''_{\text{crit}} = \text{critical heat flux (Btu/hr ft}^2\text{)}$$

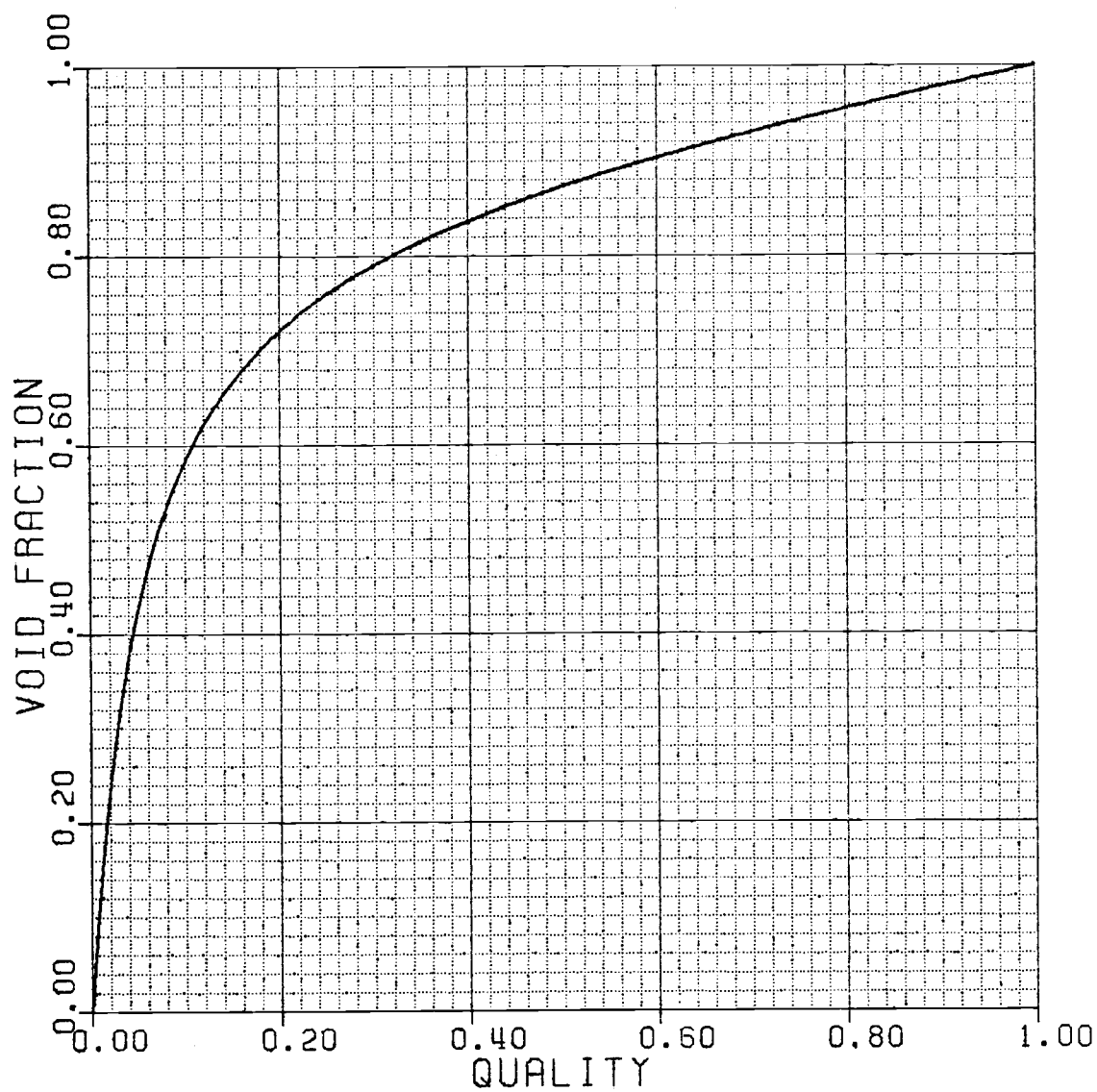


Figure 4.2 Void Fraction vs. Quality Calculated from Modified Armand Correlation at 1035 psia and 548.8° F.

X = quality

$$x_1 = 0.197 - 1.08 \times 10^{-7} \cdot G$$

$$x_2 = 0.254 - 2.6 \times 10^{-8} \cdot G$$

G = coolant mass flux (lbm/hr ft<sup>2</sup>)

At pressures other than 1000 psia,

$$q''_{crit} = q''_{crit}(\text{at } 1000 \text{ psia}) + 4.4 \times 10^5 - 4.4 \times 10^2 \cdot \text{PSIA} \quad (4-12b)$$

PSIA = system pressure (psia)

The applicable range of the parameters used above are

PSIA = 600 to 1450 psia,

G =  $4.0 \times 10^5$  to  $6.0 \times 10^6$  lbm/hr ft<sup>2</sup>,

X = negative to 0.45,

D<sub>e</sub> = 0.125 to 0.6 in. (equivalent diameter of channel),

L = 29. to 108. in (channel length).

The local quality is calculated using equation 4-10. The coolant mass flux in each channel is calculated from the active coolant mass flow rate and the active channel flow area.

$$G(I,J) = \frac{F(I,J) \cdot (144 \text{ in}^2/\text{ft}^2)}{\text{AREA}} \quad (4-13)$$

F(I,J) = mass flow rate calculated from equation 4-5

AREA = active flow area per channel (in<sup>2</sup>)

The minimum critical heat flux ratio at each node is calculated from

$$\text{MCHFR}(I,J,K) = \frac{q''_{crit}(I,J,K)}{q''(I,J,K)} \quad (4-14)$$

q''(I,J,K) = local heat flux (Btu/hr ft<sup>2</sup>)

The heat flux used above is evaluated from the relative power of the

node and three input quantities.

$$q''(I,J,K) = q''_{av} \cdot \text{PEAKF} \cdot \frac{\text{PCTPWR}}{100} \cdot P(I,J,K) \quad (4-15)$$

$q''_{av}$  = core average heat flux at rated power  
(Btu/hr ft<sup>2</sup>) (input)

PEAKF = heat flux peaking factor (input)

PCTPWR = reactor power in percent of rated power (input)

$P(I,J,K)$  = relative power of node (I,J,K)

The peak heat flux in a node will be slightly higher than the node average heat flux. Among other reasons, this may be due to fuel rods of varying enrichments within an assembly. PEAKF is intended primarily as a correction factor to account for this. Safety margins may also be included in PEAKF.

If the MCHFR calculation is requested, the calculation will be done at the beginning and end of each exposure step. The MCHFR and the indices of the node where it occurs will be printed. An edit of the MCHFR in each channel may also be requested.

## 5.0 HALING SOLUTION

The Haling solution option calculates the optimal power distribution for an operating cycle. This is determined from the beginning of cycle fuel loading description and the cycle length. The optimal power distribution is here defined as that which minimizes power peaking throughout the cycle.

The underlying principle of the Haling solution is that power peaking is minimized by a power distribution that does not change during the cycle.<sup>6</sup> It is assumed that fuel reactivity is a decreasing function of exposure near the end of the cycle. This may not be true at low exposures in fuel characterized by a high breeding ratio or containing burnable poisons. It is a generally good assumption in LWR's for the burnups encountered at the end of an operating cycle.

The Haling logic can be easily understood with the aid of figure 5.1. Curve A is postulated as the optimal reactor power distribution. Curve B has a lower power peaking ratio. This can occur only as the result of reduced reactivity, implying greater exposure, in the area of the peak. The higher exposure, however, is inconsistent with the lower peak. It implies that another power distribution, with a higher power peak, was used earlier in the cycle. Thus, power peaking can only be minimized if the power distribution and end of cycle exposure distribution are self consistent.

The Haling solution is of use in fuel reload design as well as operations. It provides a means of comparing fuel loading patterns on the basis of the minimum power peaking attainable with each. With a given



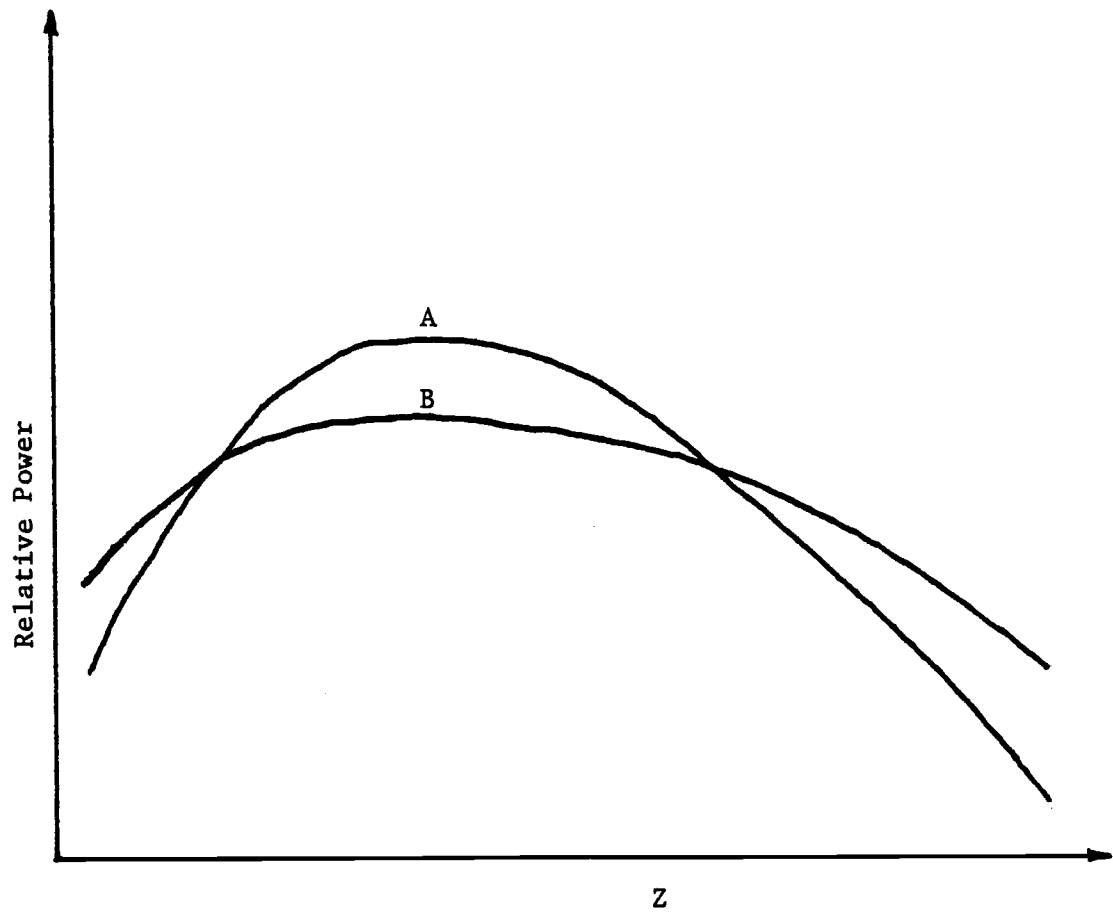


Figure 5.1 Illustration of Haling Solution Logic.

loading pattern, it can be used to determine the optimal control poison (i.e. burnable poison and control rods) distribution.

BWR control rods are very strong absorbers, producing large flux depressions in their vicinity. This makes it impractical to actually operate at the optimal power distribution. The Haling solution can, however, be used as an exposure averaged target distribution when planning rod withdrawal strategy.

The Haling solution is arrived at iteratively. Starting with an assumed power distribution, the void and end of cycle exposure distributions can be calculated. These can be used to calculate another power distribution. Due to the extremely unstable nature of the Haling iteration, an underrelaxation scheme is used.

$$P_{\ell}^H = P_{\ell}^{H-1} + \alpha_H \cdot (P_{\ell}(E^{H-1}, V^{H-1}) - P_{\ell}^{H-1}) \quad (5-1)$$

$P_{\ell}^H$  = power at node  $\ell$  for iterant  $H$

$P_{\ell}(E^{H-1}, V^{H-1})$  = power at node  $\ell$  calculated using the exposure and void distributions consistent with the previous power iterant

$H$  = Haling iteration counter

$\alpha_H$  = relaxation factor (=0.1)

The exposure for each Haling iterant is calculated from

$$E_{\ell}^H = E_{\ell 0} + P_{\ell}^H \cdot BU \quad (5-2)$$

$E_{\ell}^H$  = exposure at node  $\ell$  for iterant  $H$

$E_{\ell 0}$  = beginning of cycle exposure at node  $\ell$

$BU$  = cycle length (MWD/MTU)

## 6.0 RESULTS

In this chapter, the results of test calculations used to demonstrate the model are presented. These include benchmarking tests used to check the neutronics model, and power and void distribution tests which demonstrate feedback effects, control rod effects, and the Haling solution option. Input for the cases described in sections 6.3 and 6.4 is basically the same as that given at the end of Appendix A.

### 6.1 Benchmarking of the Neutronics Model

A 9x9x9 node test case was used to benchmark the neutronics model. Fifteen centimeter mesh widths were used and all nodes were assigned the cross sections given in table 6.1. Various sets of boundary conditions were applied to the six boundary faces of the cubical model. The calculated flux distributions could then be compared to analytical solutions in order to verify the results.

Table 6.1 Cross Sections Used in Benchmarking Cases.

	D (cm)	$\Sigma_a$ (cm <sup>-1</sup> )	$\nu\Sigma_f$ (cm <sup>-1</sup> )	$\kappa\Sigma_f$ (Mev/cm)	$\Sigma_r$ (cm <sup>-1</sup> )
Fast	1.418	$0.7104 \times 10^{-2}$	$0.4074 \times 10^{-2}$	0.3231	$0.1038 \times 10^{-1}$
Thermal	0.5492	$0.7522 \times 10^{-1}$	0.1069	9.020	

The eleven cases which were run are summarized in table 6.2. Case 1 represented a reactor of infinite dimensions. A flat flux distribution was calculated as expected. The k-effective for this case was calculated as 1.1938 - the same as k-infinity calculated directly from the two-group cross sections.

Table 6.2 Neutronics Model Benchmarking Cases.

<u>Case #</u>	<u>Description</u>	<u>Boundary Conditions</u>	<u>Analytical Solution</u>
1	Infinite reactor	zero current on all 6 faces	$\phi(x,y,z) = C$
2	Cubical reactor 135x135x135 cm	zero flux on all 6 faces	$\phi(x,y,z) = C \cdot \sin(x\pi/a) \cdot \sin(y\pi/a) \cdot \sin(z\pi/a)$
3	Semi-infinite slab, 135 cm thick	zero flux faces 1 & 2, zero current faces 3,4,5, &6	$\phi(x,y,z) = C \cdot \sin(x\pi/a)$
4	Same as #3	zero flux faces 3 &4, zero current faces 1,2,5, &6	$\phi(x,y,z) = C \cdot \sin(y\pi/a)$
5	Same as #3	zero flux faces 5 &6, zero current faces 1,2,3, &4	$\phi(x,y,z) = C \cdot \sin(z\pi/a)$
6	Semi-infinite slab, 270 cm thick	zero flux face 1, zero current faces 2,3,4,5, &6	$\phi(x,y,z) = C \cdot \sin(x\pi/2a)$
7	Same as #6	zero flux face 2, zero current faces 1,3,4,5, &6	$\phi(x,y,z) = C \cdot \cos(x\pi/2z)$
8	Same as #6	zero flux face 3, zero current faces 1,2,4,5, &6	$\phi(x,y,z) = C \cdot \sin(y\pi/2a)$
9	Same as #6	zero flux face 4, zero current faces 1,2,3,5, &6	$\phi(x,y,z) = C \cdot \cos(y\pi/2a)$
10	Same as #6	zero flux face 5, zero current faces 1,2,3,4, &6	$\phi(x,y,z) = C \cdot \sin(z\pi/2a)$
11	Same as #6	zero flux face 6, zero current faces 1,2,3,4, &5	$\phi(x,y,z) = C \cdot \cos(z\pi/2a)$

C = constant

a = 135.

Case number 2 represented a cubical reactor. The x-y flux distribution calculated at the center plane is shown in figure 6.1. Figure 6.2 shows a graph of the flux distribution through the center of this plane. Calculated values are shown as small circles and the analytic solution is represented by a line.

Cases 3 through 5 represented a 135 centimeter thick semi-infinite slab reactor. A 270 centimeter semi-infinite slab reactor was represented in cases 6 through 11. The case descriptions and results were essentially identical except for orientation and slab thickness. The resulting transverse flux distributions are shown in figures 6.3 and 6.4. Again, calculated values are shown as circles and the analytic solutions are represented by lines.

$\phi$

0.03	0.09	0.13	0.16	0.17	0.16	0.13	0.09	0.03
0.030	0.087	0.133	0.163	0.174	0.163	0.133	0.087	0.030
0.09	0.25	0.38	0.47	0.50	0.47	0.38	0.25	0.09
0.087	0.250	0.383	0.470	0.500	0.470	0.383	0.250	0.087
0.13	0.38	0.59	0.72	0.77	0.72	0.59	0.38	0.13
0.133	0.383	0.587	0.720	0.766	0.720	0.587	0.383	0.133
0.16	0.47	0.72	0.88	0.94	0.88	0.72	0.47	0.16
0.163	0.470	0.720	0.883	0.940	0.883	0.720	0.470	0.163
0.17	0.50	0.77	0.94	1.00	0.94	0.77	0.50	0.17
0.174	0.500	0.766	0.940	1.000	0.940	0.766	0.500	0.174
0.16	0.47	0.72	0.88	0.94	0.88	0.72	0.47	0.16
0.163	0.470	0.720	0.883	0.940	0.883	0.720	0.470	0.163
0.13	0.38	0.59	0.72	0.77	0.72	0.59	0.38	0.13
0.133	0.383	0.587	0.720	0.766	0.720	0.587	0.383	0.133
0.09	0.25	0.38	0.47	0.50	0.47	0.38	0.25	0.09
0.087	0.250	0.383	0.470	0.500	0.470	0.383	0.250	0.087
0.03	0.09	0.13	0.16	0.17	0.16	0.13	0.09	0.03
0.030	0.087	0.133	0.163	0.174	0.163	0.133	0.087	0.030

$\phi$

calculated

analytic  
solution

Figure 6.1 Comparison of Calculated and Analytical Solutions of the Relative Flux Distribution for the Center Plane of a Cubical Reactor.

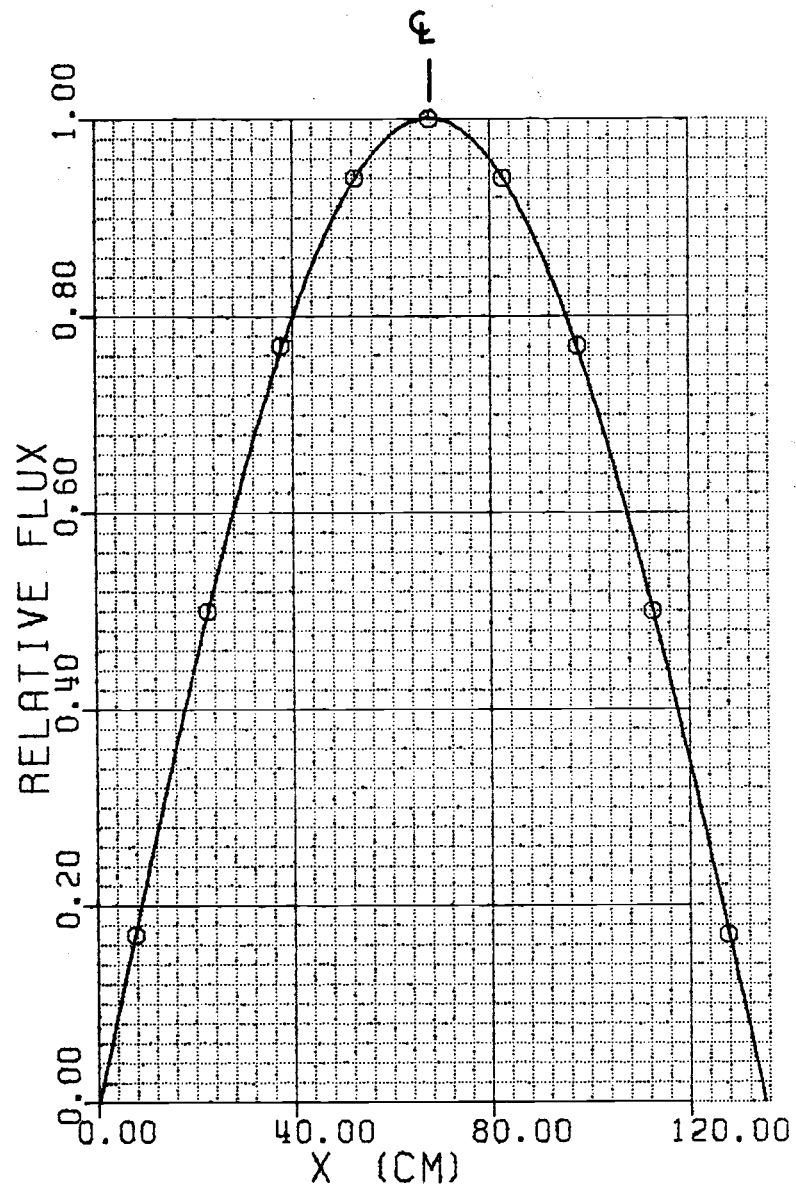


Figure 6.2 Comparison of Calculated and Analytical Solutions of the Relative Flux Profile through the Center of a Cubical Reactor.

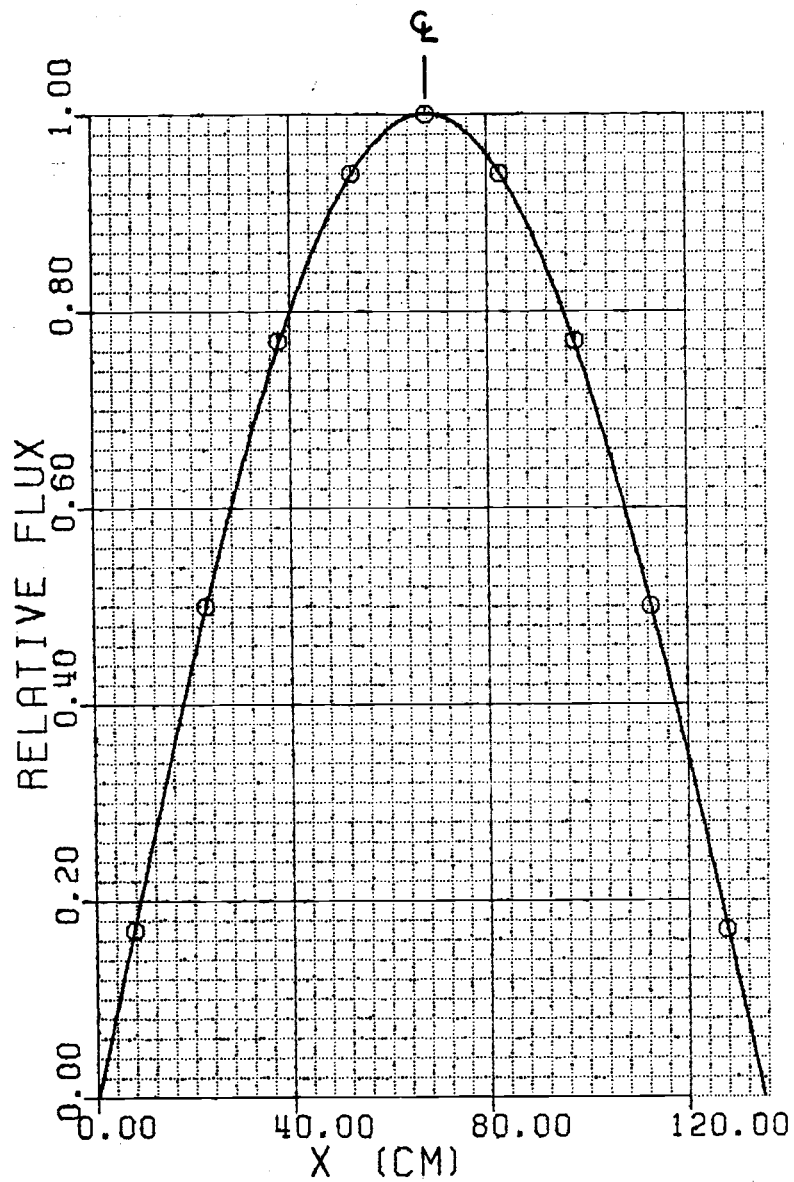


Figure 6.3 Comparison of Calculated and Analytical Solutions of the Relative Flux Distribution in a 135 cm Semi-Infinite Slab Reactor.



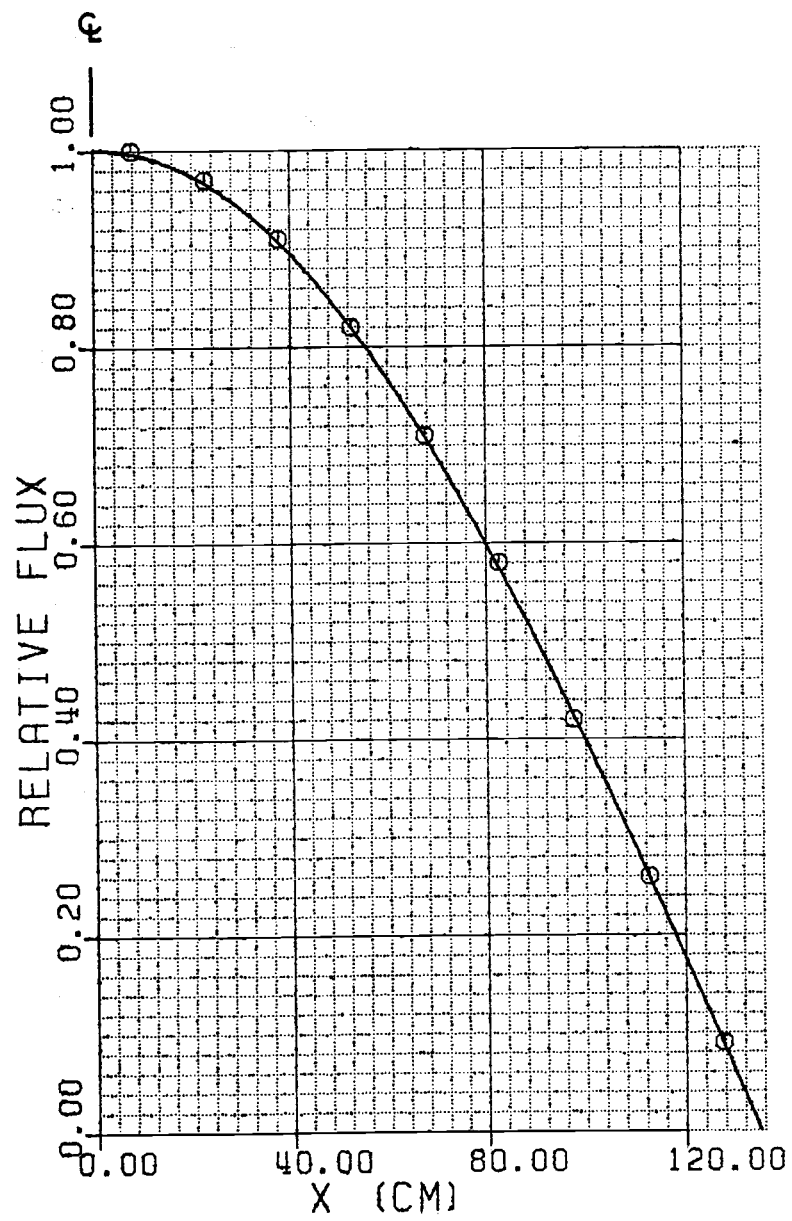


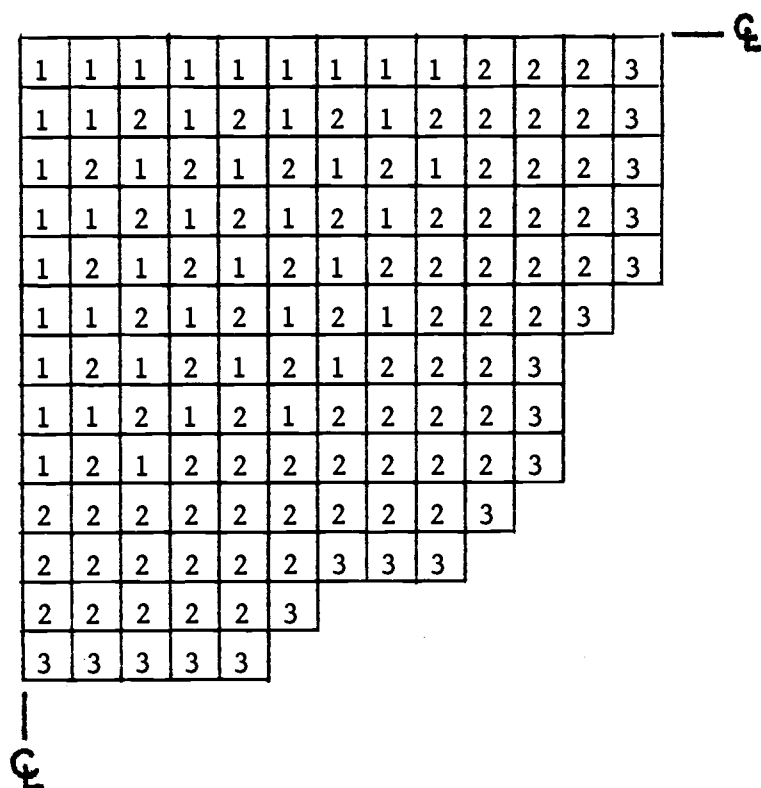
Figure 6.4 Comparison of Calculated and Analytical Solutions of the Relative Flux Distribution in a 270 cm Semi-Infinite Slab Reactor.

## 6.2 Reference Reactor

The Edwin Hatch Unit #2 reactor was used as the reference reactor for the following calculations. This is a boiling water reactor of recent design with a thermal output of 2436 megawatts. The cycle 1 fuel loading pattern was used and is shown in figure 6.5. Other pertinent information can be found in table 6.3<sup>13</sup>.

BWR fuel assemblies commonly use more than one fuel pin enrichment in a single assembly as well as axial burnable poison distributions. This is the case with the core which was modeled. Due to limited resources it was not feasible to properly generate two-group input for use in the test calculations. A two dimensional depletion calculation, used to generate exposure dependent assembly averaged cross sections, could not be done. The burnable poison (gadolinium) distributions used were not available due to their proprietary nature, nor were cross sections for gadolinium. The approach taken was to generate cross sections for each assembly type with LEOPARD, using assembly average enrichments and volume fractions. Gadolinium was ignored. These restrictions limit the validity of the results obtained.

The control rod worths used are shown in table 6.4. These are typical rod worths for a BWR. It was not possible to obtain more accurate values. The rod worths of table 6.4 were used for all three fuel types at all exposures.



Region	Average Enrichment w/o U-235
1	1.83
2	2.33
3	0.711

Figure 6.5 Edwin I. Hatch Nuclear Plant Unit #2 Fuel Loading Pattern.

Table 6.3 Design Data for Edwin I. Hatch Nuclear Plant Unit #2.

Nominal thermal output	2436	Mw
Incore coolant flow rate	$7.70 \times 10^7$	lbm/hr
Nominal system pressure	1035.	psia
Coolant saturation temperature	548.8	$^{\circ}\text{F}$
Average heat flux	145,060.	Btu/hr ft <sup>2</sup>
Core inlet enthalpy	526.9	Btu/lb
Active coolant flow area per assembly	15.82	in <sup>2</sup>
Active fuel length	150.	in
Assembly lattice	8x8	
Assembly pitch	6.0	in

Table 6.4 Control Rod Worths.

<u>Void Fraction</u>	<u>Rod Worth ( <math>\Delta k/k</math> )</u>
0.00	0.24
0.32	0.27
0.64	0.32

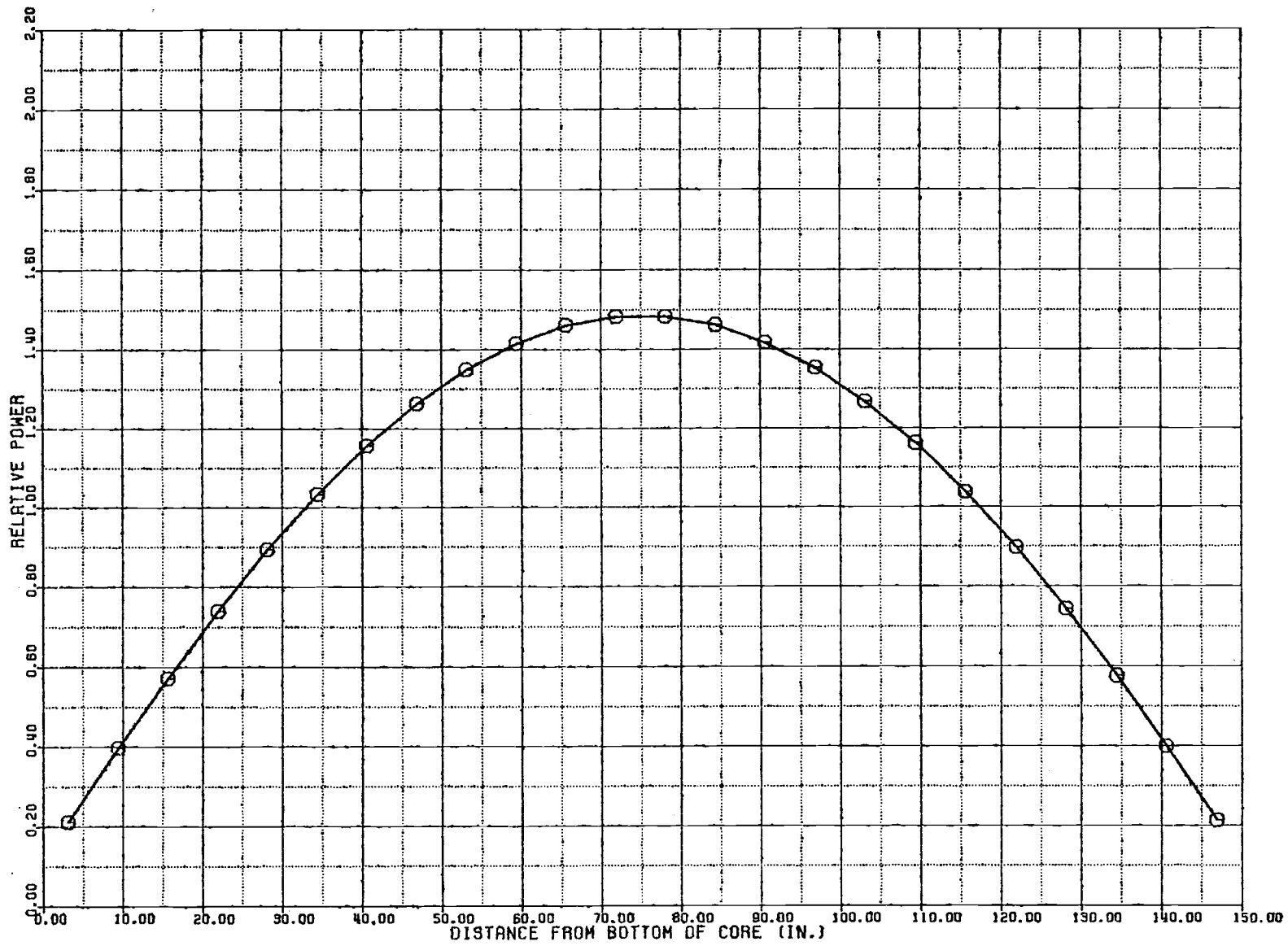
### 6.3 Power and Void Distributions

The effect of voiding on the axial power distribution is shown in figures 6.6 through 6.8. All three cases illustrated were run at beginning of life, nominal flow conditions with control rods completely withdrawn (ARO). At hot zero power (HZIP), the distribution has a cosine shape as expected. At 50% power, coolant voiding decreases moderation in the upper core. This shifts the power distribution downward where the core is relatively more reactive. The increased voiding at hot full power (HFP) shifts the distribution even lower and results in a higher power peak. Control rods are used to offset this void induced tendency toward power peaking in the lower part of the core.

Control rod effects are illustrated in figure 6.9. These cases were run at beginning of life, hot full power conditions. Inserting all rods half way in the core shifts the power peak to the upper part of the core. The downward shift due to voiding is still evident, but the control rods have almost completely eliminated power production in the lower half of the core. Completely inserting all rods gives a very large peak at the bottom of the core. This case is unrealistic in that it is subcritical, but it does demonstrate the effect of increased control rod worths with increasing void fraction.

Three control rod pattern cases are described by figures 6.10 through 6.12. These were run at hot full power, beginning of life conditions. The resulting axial power distributions are shown in figure 6.13. The effects illustrated by the three previous cases are still evident, but have been complicated by varying rod positions. Deeply

Figure 6.6 Axial Power Distribution, HZP, ARO.



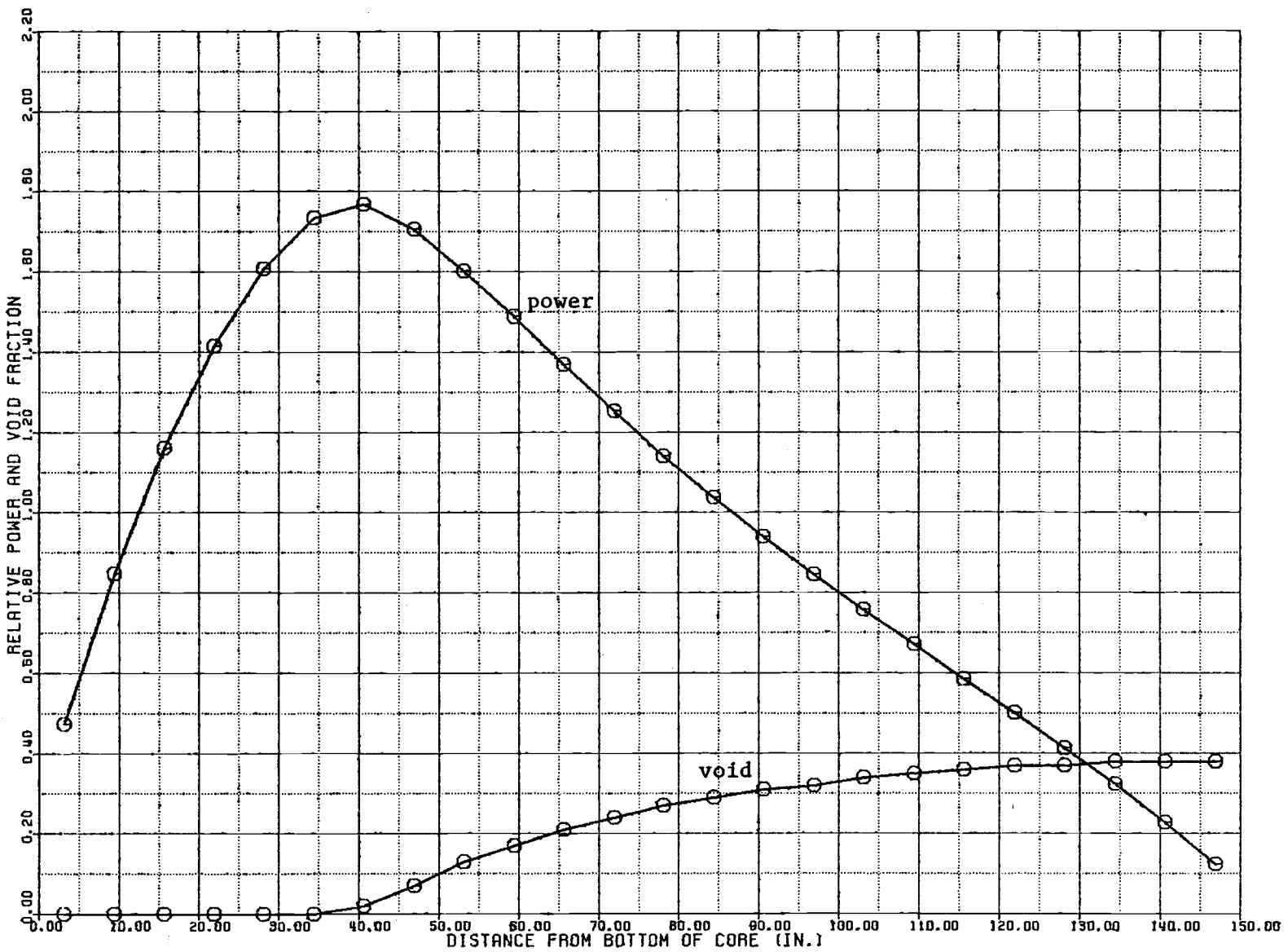


Figure 6.7 Axial Power and Void Fraction Distributions at 50% Power, ARO.

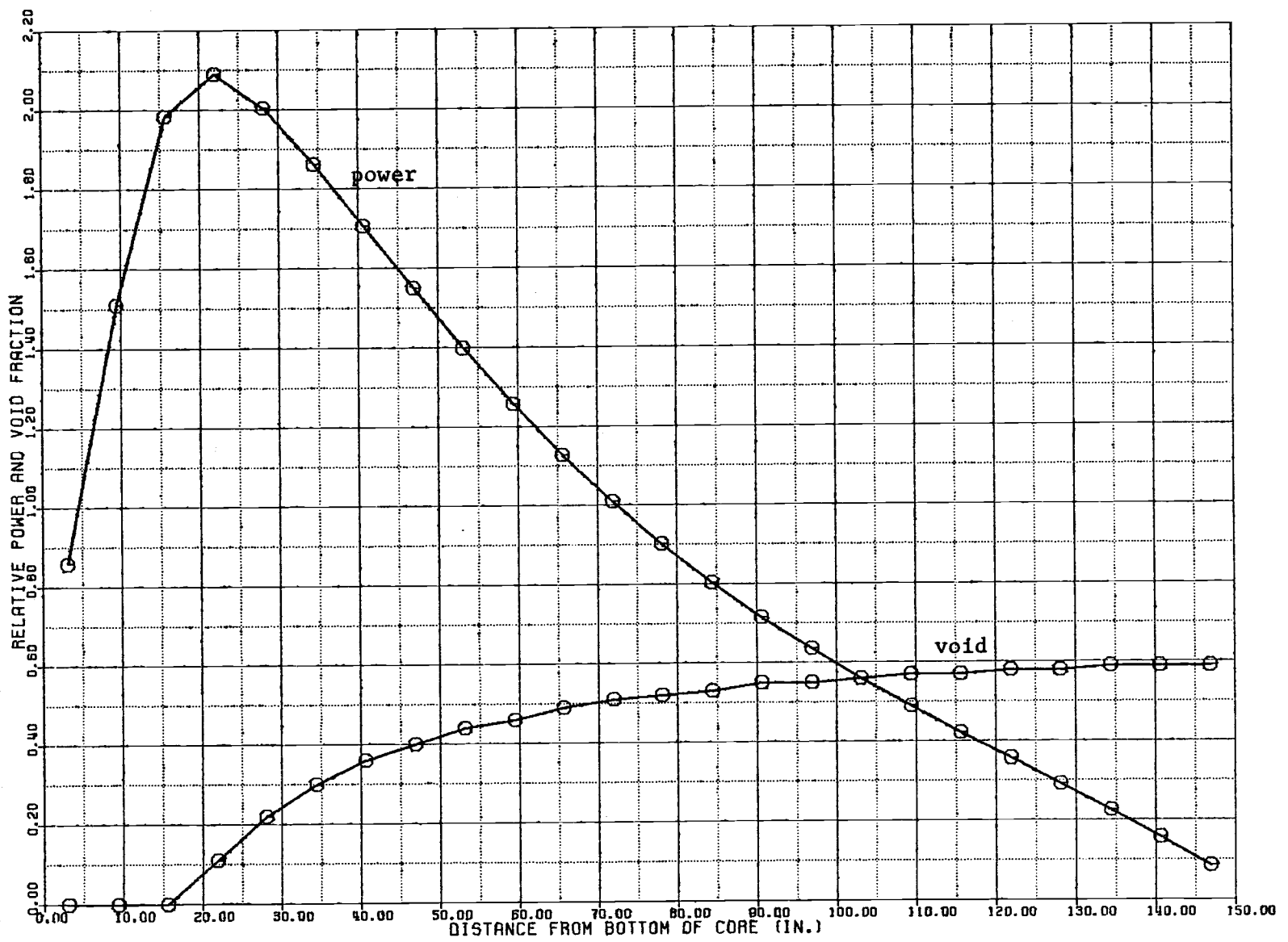
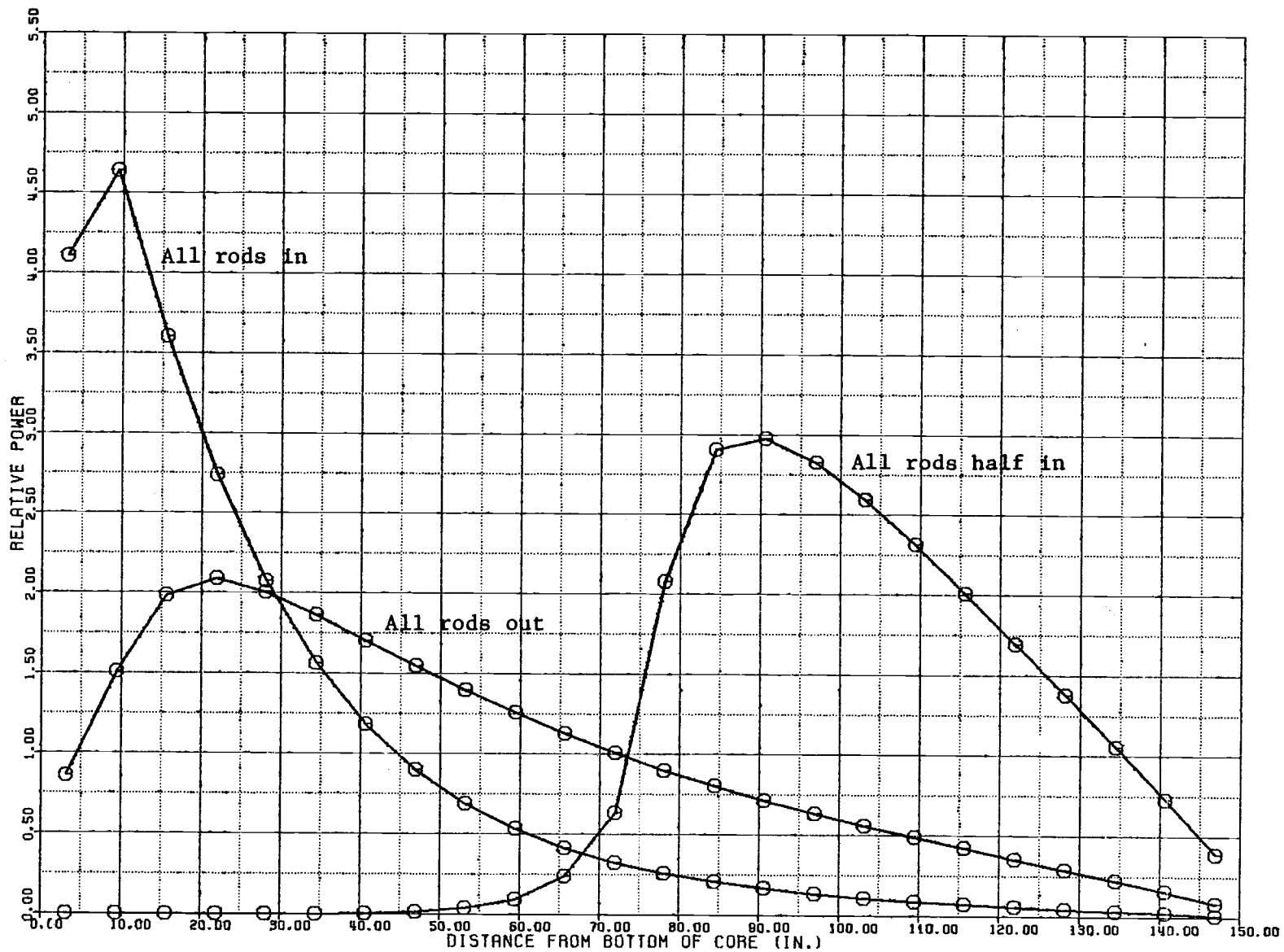


Figure 6.8 Axial Power and Void Fraction Distributions at Hot Full Power, ARO.



Figure 6.9 Illustration of Control Rod Effects on the Axial Power Distribution.



Nodes Inserted

24	0	24	0	24	0	12
0	8	0	8	0	8	0
24	0	24	0	24	0	0
0	8	0	8	0	8	
24	0	24	0	24	0	
0	8	0	8	0		
12	0	0				

$$k_{\text{eff}} = 1.0817$$

$$\text{Peak} = 3.242$$

$$\text{Radial Peak} = 1.455$$

$$\text{Axial Peak} = 2.088$$

Figure 6.10 Control Rod Positions for Control Rod Pattern Case #1.

Nodes Inserted

2	24	16	24	2	24	12
24	16	24	2	24	16	0
16	24	2	24	16	24	0
24	2	24	16	24	2	
2	24	16	24	2	0	
24	16	24	2	0		
12	0	0				

$$k_{\text{eff}} = 0.9764$$

$$\text{Peak} = 5.528$$

$$\text{Radial Peak} = 1.888$$

$$\text{Axial Peak} = 1.753$$

Figure 6.11 Control Rod Positions for Control Rod Pattern Case #2.

Nodes Inserted

23	0	23	0	23	0	12
0	16	0	16	0	16	0
23	0	23	0	23	0	0
0	16	0	16	0	16	
23	0	23	0	23	0	
0	16	0	16	0		
12	0	0				

$$k_{\text{eff}} = 1.0598$$

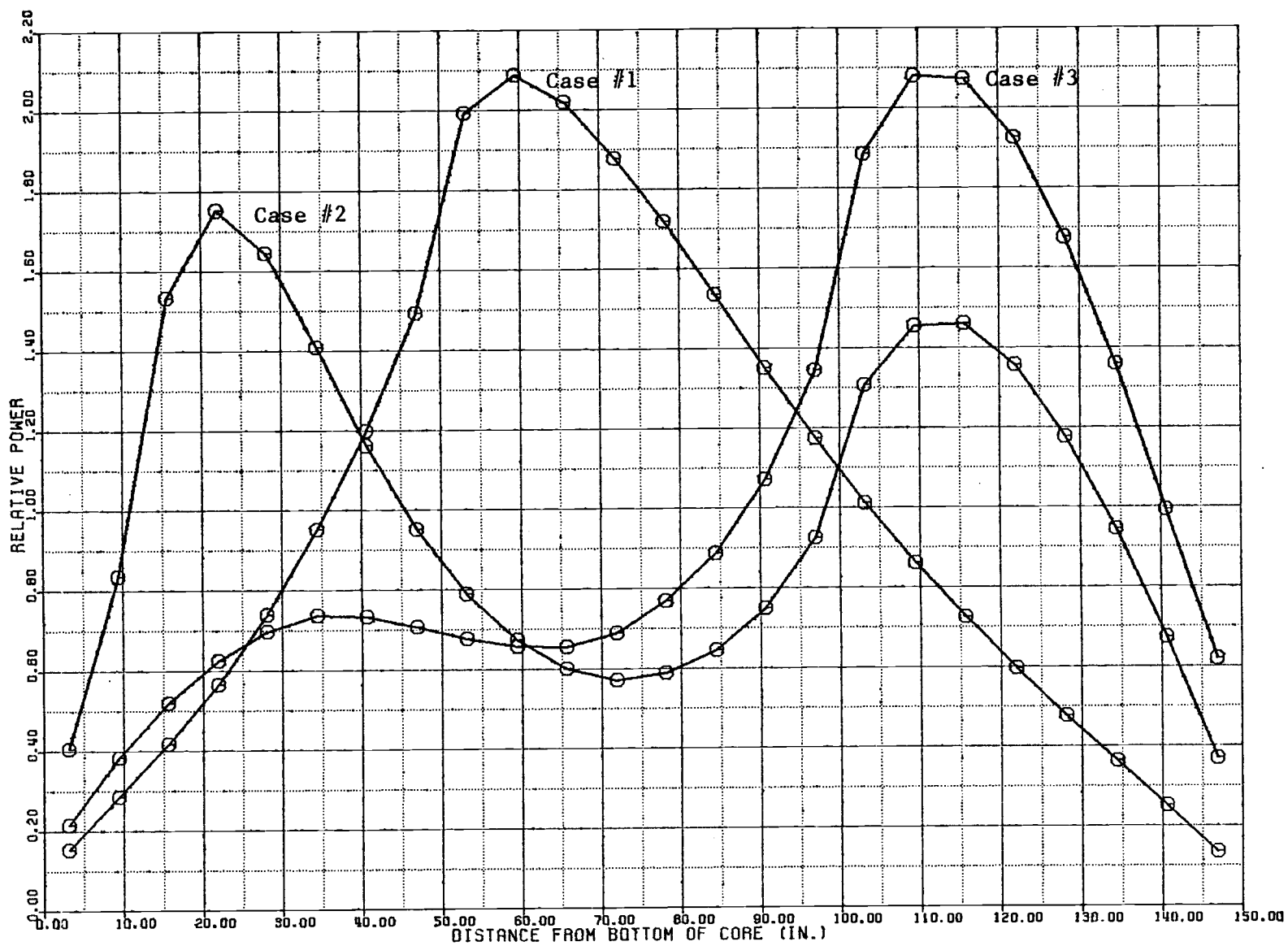
$$\text{Peak} = 3.307$$

$$\text{Radial Peak} = 1.496$$

$$\text{Axial Peak} = 2.081$$

Figure 6.12 Control Rod Positions for Control Rod Pattern Case #3.

Figure 6.13 Axial Power Distributions Calculated from Control Rod Pattern Cases.



inserted rods coupled with coolant voiding tend to produce a peak near the bottom of the core. Another power peak occurs just above the tip of partially inserted rods. These effects make it difficult to achieve a relatively flat power distribution. The axially integrated two-dimensional power distributions are given in figure 6.14.

A beginning of life axial power distribution given in the Hatch Unit #2 Final Safety Analysis Report is shown in figure 6.15<sup>13</sup>. Control rod positions and axial burnable poison distributions for this case are not specified. This is a much flatter distribution than those obtained in the previously described cases. Several attempts to produce a comparable distribution were unsuccessful. Presumably the difficulty was due to improper input, particularly due to the fact that burnable poison distributions were not represented.

Figure 6.14 Radial Power Distributions Calculated from Control Rod Pattern Cases.

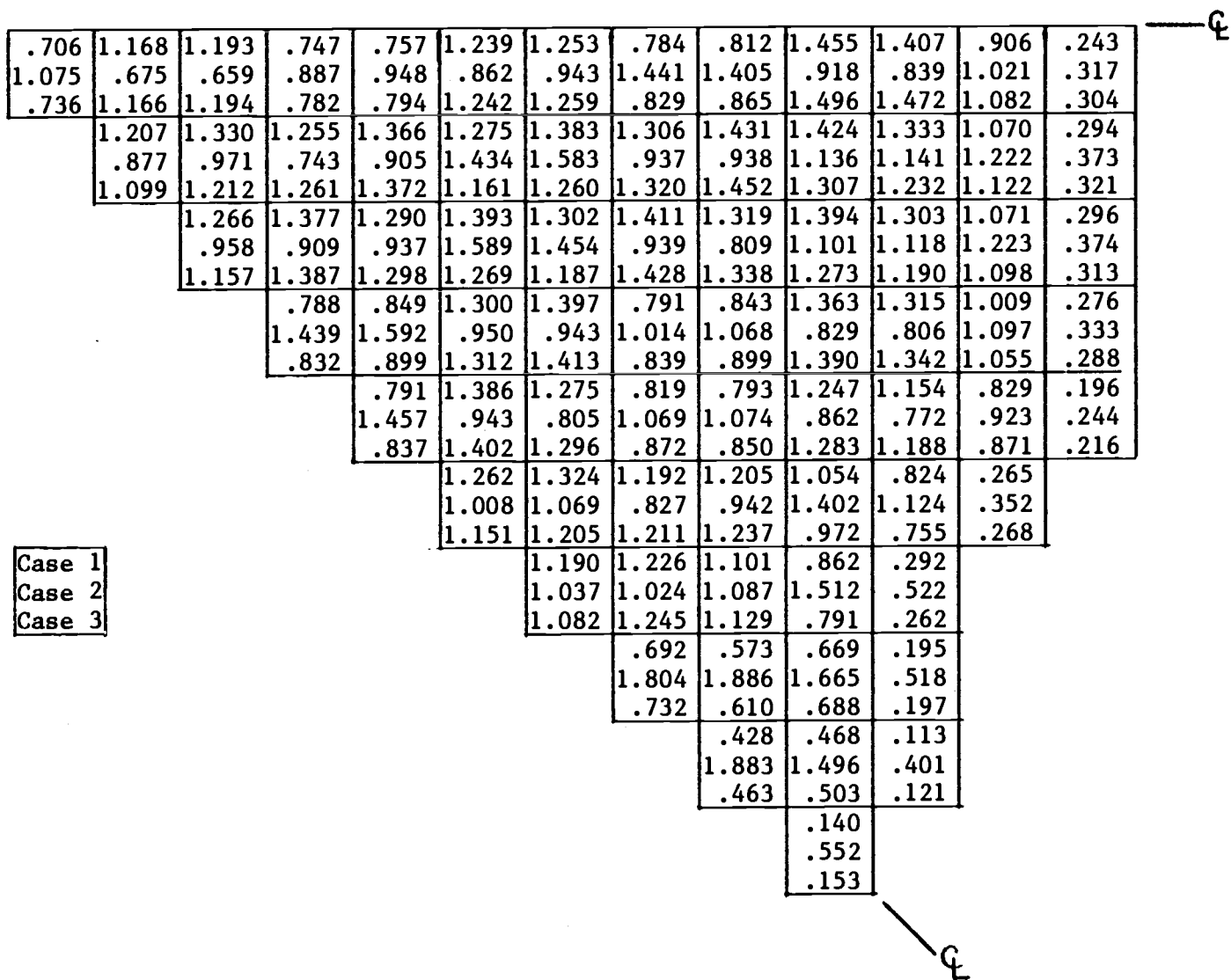
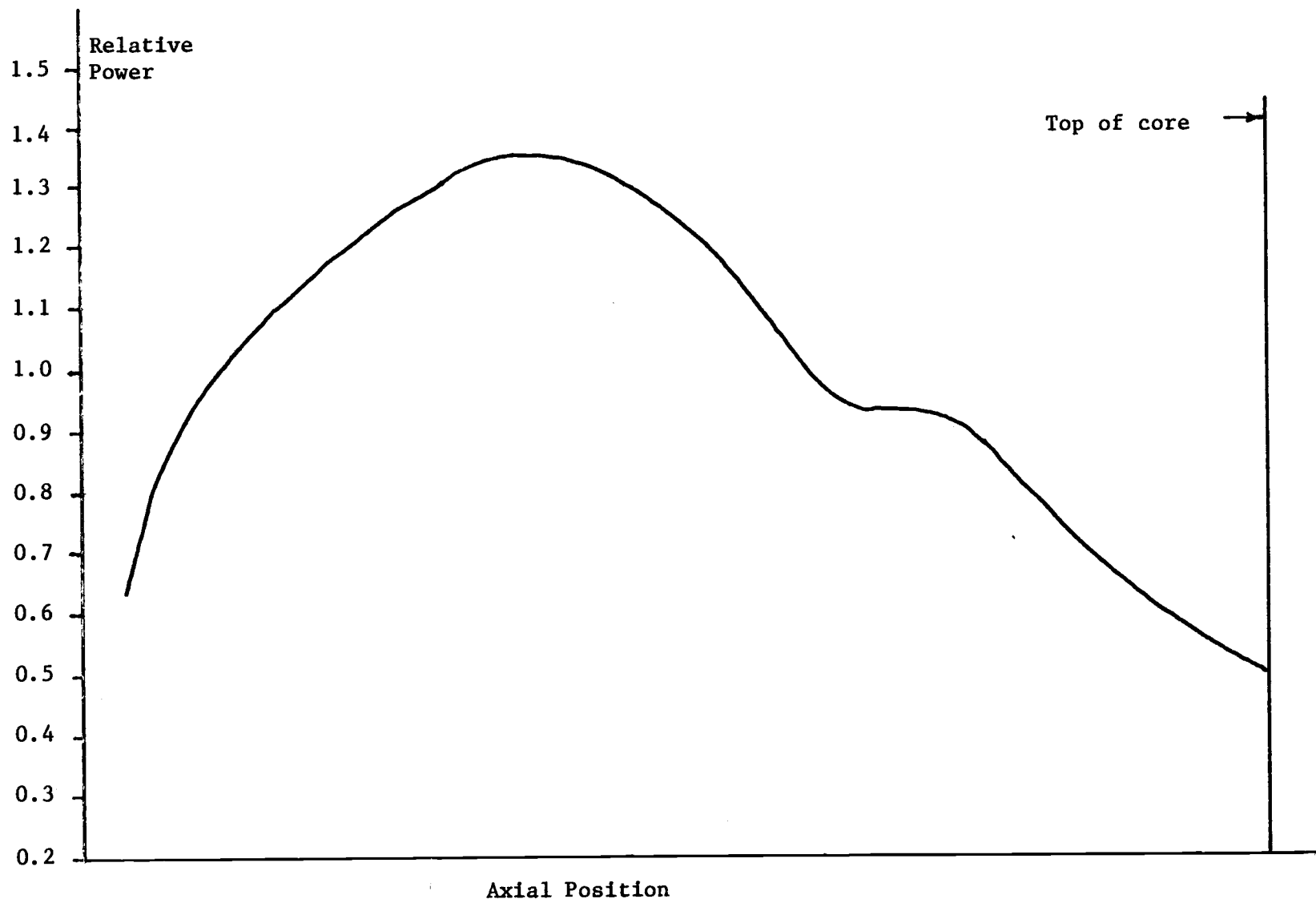


Figure 6.15 Axial Power Distribution from Edwin I. Hatch Nuclear Plant Unit #2 FSAR.





#### 6.4 Haling Solution

A Haling solution calculation was run using an end of cycle average core exposure of 12000 MWD/MTU. The resulting x-y power distribution is shown in figure 6.16. Figure 6.17 shows the axial power and void distributions obtained. Judging from the control rod cases, it would be impossible to achieve this power distribution at beginning of life conditions without an axial distribution of burnable poison. As expected, a very flat power distribution was calculated with a very low peak to average power ratio. The peak to average ratio was 1.396 for this case.

Figure 6.18 shows a Haling axial power distribution for a typical BWR. The calculated power distribution in figure 6.17 compares very favorably with this.

Figure 6.16 Radial Power Distribution from Haling Solution Case.

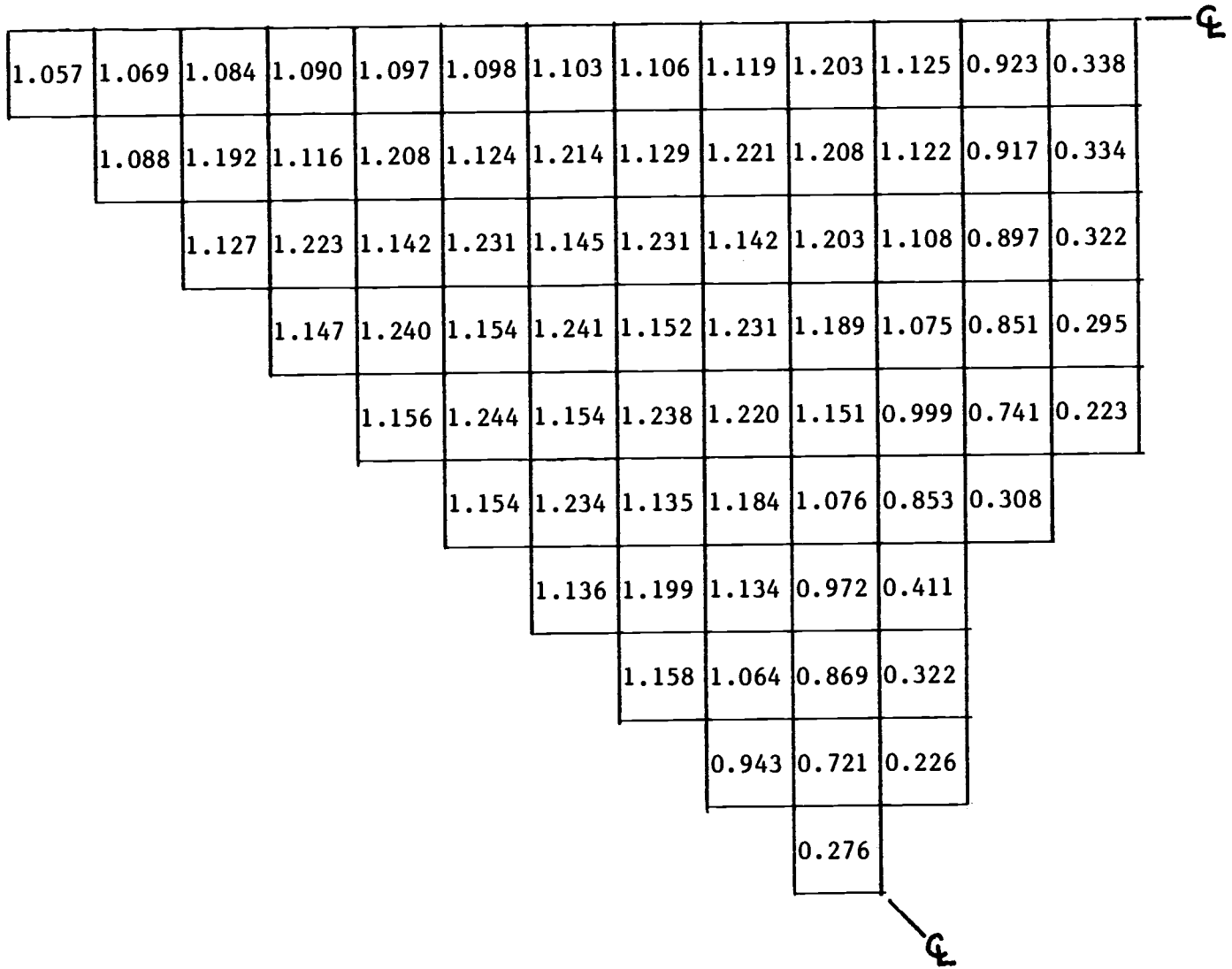


Figure 6.17 Axial Power and Void Fraction Distributions from Haling Solution Case.

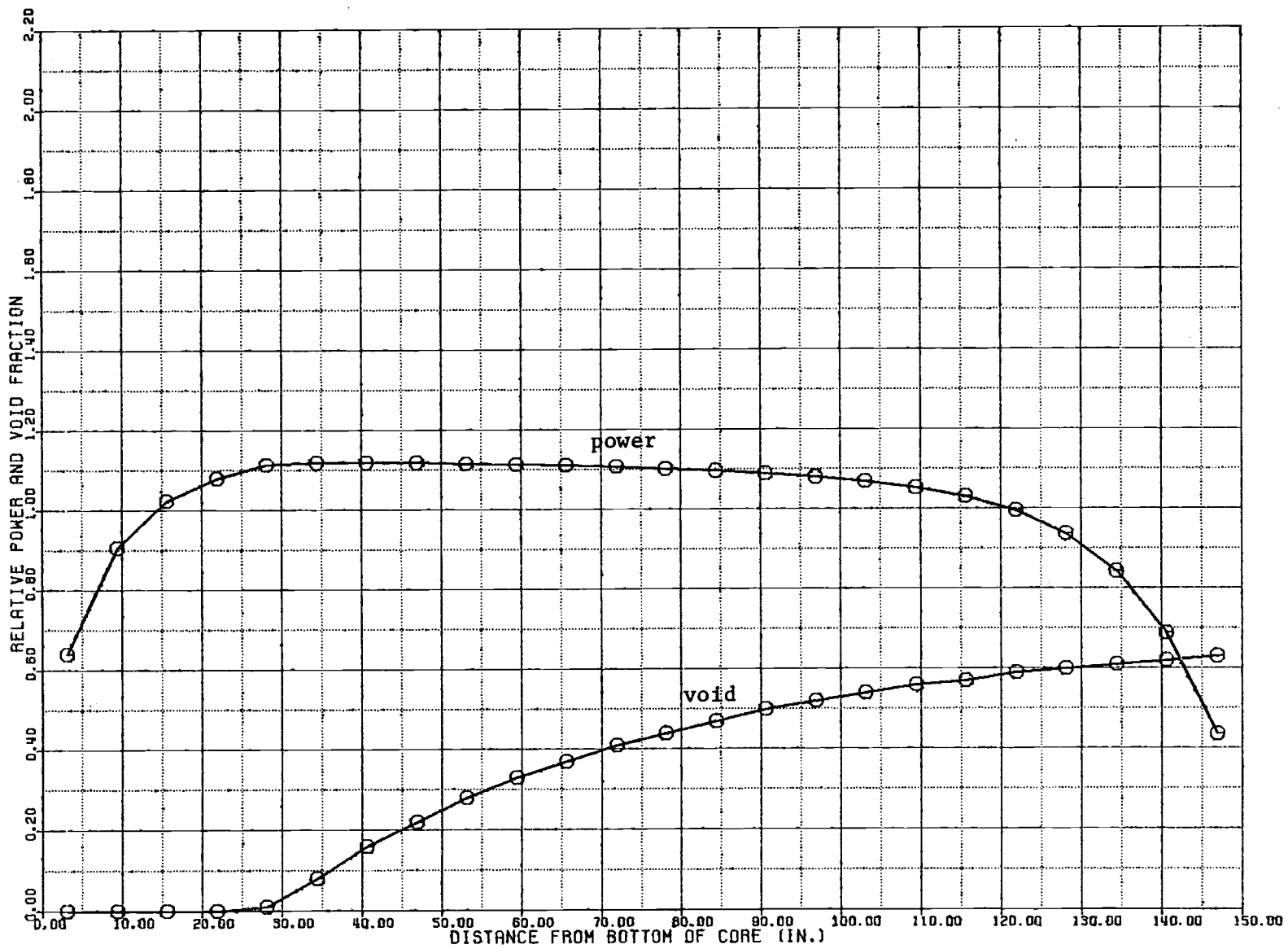
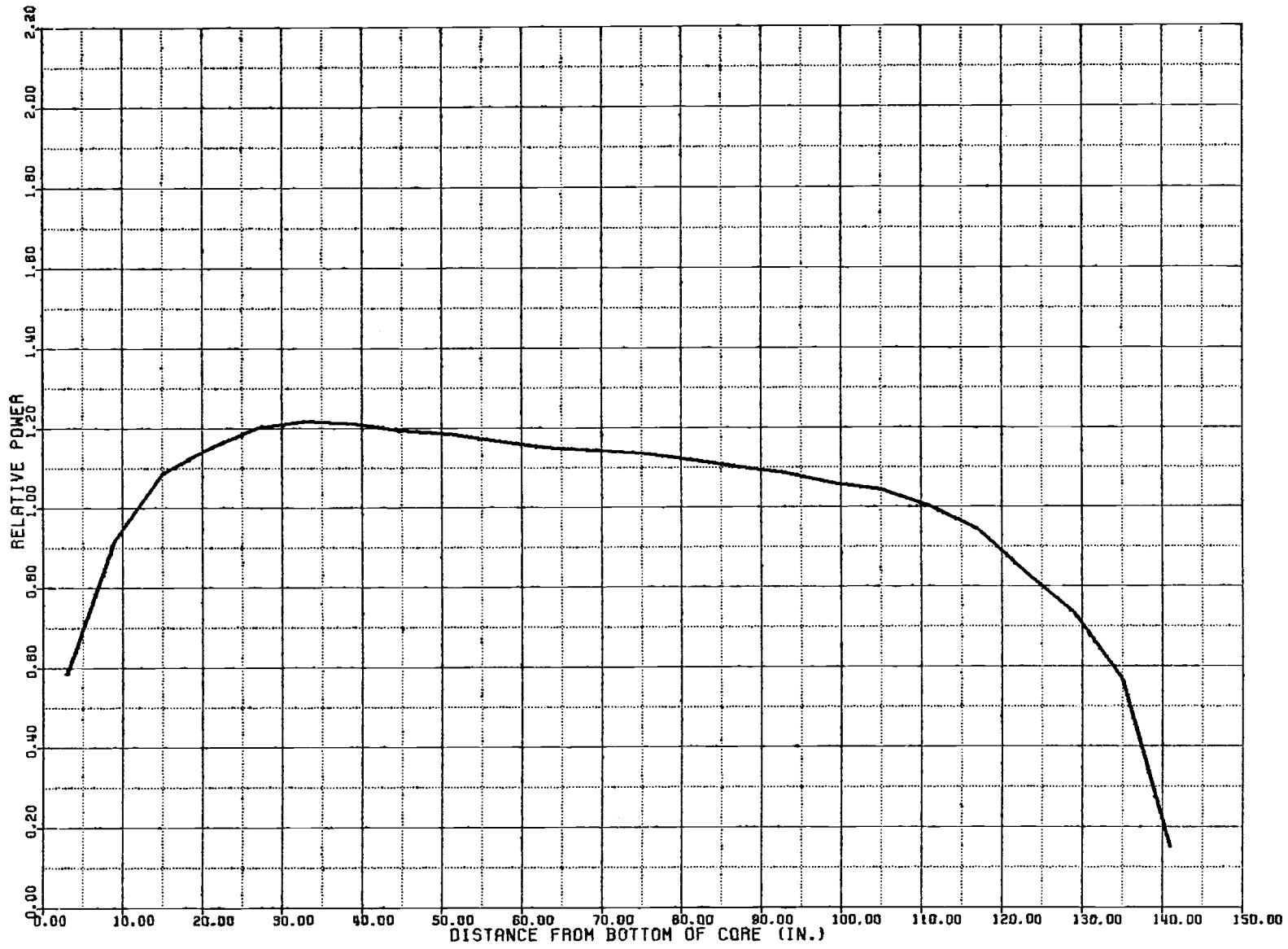


Figure 6.18 Typical Halving Axial Power Distribution.



## 7.0 CONCLUSIONS

Test results indicate that the computer code which has been developed is capable of modeling the burnup dependent static behavior of a boiling water reactor with reasonable accuracy. Benchmarking tests of the neutronics model yielded results that agreed very well with analytical solutions. Control rod pattern tests did not give axial power distributions as flat as had been expected. This was presumably due to the absence of axial burnable poison distributions. It is also likely that better rod patterns would have improved these results. The determination of optimal rod patterns, however, was beyond the scope of this thesis. The end of cycle power distribution calculated by the Haling solution test showed close agreement with the expected distribution. Problems with control rods and burnable poisons would not affect the Haling solution results since burnable poisons are depleted and control rods are fully withdrawn at the end of an ideal operating cycle.

Unfortunately, a more rigorous test of the integrated model was not possible. A complete description of the nuclear design of the reference reactor's core was not obtainable due to the proprietary nature of part of the information. Cross section input could not be properly generated due to the unavailability of a PDQ type code. Cross sections suitable for test calculations were generated using the zero-dimensional LEOPARD code.

Control rods and burnable poisons can be used to shape the power distribution in order to improve core performance. The optimal power distribution is generally defined as that which minimizes power peaking

throughout an operating cycle and is given by the Haling solution. The control rod effects which have been demonstrated indicate that the Haling power distribution cannot be attained with control rods. As the rods are withdrawn to counteract fuel depletion, the fuel exposure distribution becomes the dominant influence on the power distribution. The problem that remains is how to select rod withdrawal patterns which, while minimizing power peaking and maintaining the core reactivity within flow control bounds, will yield an end of cycle exposure distribution consistent with the Haling power distribution.

With the proper input, the code which has been developed should provide a reasonable model of a boiling water reactor core. Such a model can be used to develop control rod withdrawal and fuel loading strategies which can be applied to improve reactor performance. A list of possible future modifications which can improve the accuracy and extend the applicability of the model are given below.

#### List of Future Improvements

1. Extension of the xenon model to include time dependency would enable the model to predict the effects of xenon transients.
2. The Doppler reactivity effect has not been included in the model. Increasing fuel temperature causes Doppler broadening of U-238 and Pu-240 resonance absorption peaks. This results in increased resonance absorption. The effect is worth about one-half of a percent in reactivity. Inclusion of this feedback effect should produce a slight improvement in results.
3. One of the largest reactivity effects is due to coolant voiding.

An improvement in the void fraction model could improve the accuracy of the predicted power distribution.

4. The model is limited to reactor cores with quadrant symmetry. This is sufficient for most cases of interest since control rod patterns and most fuel loading patterns have octant or quadrant symmetry. Extension to a half or full core model would require a great deal of modification to the program, but would extend its applicability.

5. The control rod model used assumes only the thermal absorption cross section changes when a node is rodded. Inclusion of separate table sets for rodded nodes may improve the predicted effects of control rods by correcting for the changes in other two-group constants.

6. The determination of optimal control rod withdrawal patterns is one of the biggest problems in BWR operation. A possible future project using this program is the development of a logic for an automated search for optimal control rod withdrawal patterns.

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## APPENDICES

## APPENDIX A

## Input Instructions

Unless otherwise noted, all input is in NAMELIST or free format. NAMELIST input is initiated with \$INPUT (&INPUT on IBM computers). The dollar sign (or &) must begin in column 2 of an 80 column record. \$INPUT is followed by a list of variable assignments in the form of

variable=value .

These are separated by commas. NAMELIST input is terminated by a dollar sign (&END on IBM) following the last variable assignment. NAMELIST variables not referenced retain their previous, or default, values.

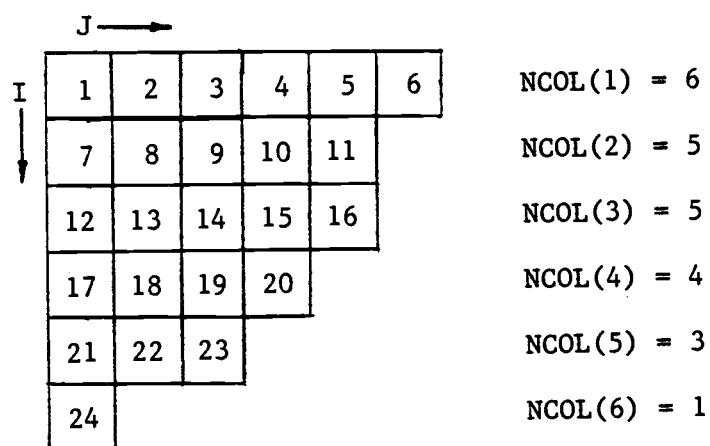
Free format input consists of a list of values separated by blanks and/or commas. These are assigned, in order, to the variables in the input list. Free format input may continue on more than one record if needed. If desired, the input scan can be terminated by a slash (/). The remaining variables will retain their previous values.

The following variables and definitions are used in the input instructions:

NODES = total number of nodes represented  
NP = number of nodes in each axial plane  
NZ = number of axial nodes represented  
NX = maximum number of nodes per row

2-D array format: In 2-D array format, NP values are input in free format. These values are assigned to planar locations in the order illustrated in figure A.1.

3-D array format: NODES values are input in free format and are assign-



NP = 24

NX = 6

Figure A.1 Quarter Core Plane Indexing Conventions.

ed to each node in the problem, plane by plane, as in 2-D array format. The first axial plane is at the bottom of the core, and the last is at the top.

Input records are cards or 80 column card images.

### Detailed Input Description

(Listed in order of occurrence)

1. Title card. FORMAT (20A4)
2. Problem description in NAMELIST format. NAMELIST name = INPUT.

Recommended values are marked by asterisks (\*).

<u>NAMELIST Variable</u>	<u>Description</u>	<u>Default</u>
IPTYPE	IPTYPE = 1 for burnup step problem. IPTYPE = 2 for Haling solution.	1
IGEOM	IGEOM = 1 for quarter core geometry. IGEOM = 2 for square plane geometry.	1
NCOL(NX)	NCOL(I), I=1, NX are the number of nodes in each hori- zontal row of a planar cross section. (see fig. A.1)	
DX	Node width in centimeters. (assembly pitch)	
DZ	Node height in centimeters.	
ICNTRL	ICNTRL = 0 if no control rods are used. ICNTRL = N if control rods are used. Control rod worths corres- ponding to N table sets will be read.	0
NBANKS	If NBANKS = 0, control rod positions will be input as a two-dimensional array giving rod positions at each planar coordinate. If NBANKS = N, rod positions will be set by banks. A rod bank is a defined set of coor-	

<u>NAMELIST</u> <u>Variable</u>	<u>Description</u>	<u>Default</u>
	dinates corresponding to a group of rods that are moved as a unit.	
IPDRD	IPDRD = 1 if a guess at the converged power distribution is to be input. This may speed convergence in some cases. If IPDRD = 0, a flat power distribution is used as the first guess.	0
IEXRD	IEXRD = 0 sets the fuel exposure at each node to 0.0 MWD/MTU. IEXRD = 1 if fuel exposures are to be input in 3-D array format. IEXRD = -1 if fuel exposures are to be input axially at selected planar coordinates. (IEXRD = -1 must be used if the exposure punch from a previous case is to be read.)	0
IEDIT(1)	IEDIT(1) = -1 if 3-D and integrated power distributions are to be edited. IEDIT(1) = 0 if only integrated (2-D) power distributions are to be edited. IEDIT(1) = 1 if only 3-D power distributions are to be edited.	-1
IEDIT(2)	IEDIT(2) = -1 if 3-D and assembly average exposure distributions are to be edited. IEDIT(2) = 0 if only assembly average exposure distributions are to be edited. IEDIT(2) = 1 if only 3-D exposure distributions are to be edited.	0
IEDIT(3)	IEDIT(3) = -1 to punch the exposure distribution at the end of each burnup step. IEDIT(3) = 0 for no exposure punch. IEDIT(3) = 1 to punch the exposure	0

<u>NAMelist</u> <u>Variable</u>	<u>Description</u>	<u>Default</u>
	distribution at the end of the case.	
IEDIT(4)	IEDIT(4) = -1 to edit void fractions for the top plane only. IEDIT(4) = 0 for no void fraction edit. IEDIT(4) = 1 to edit 3-D void fraction distributions.	1
IEDIT(5)	IEDIT(5) = -1 to edit the location and value of the MCHFR. IEDIT(5) = 0 for no MCHFR edit. IEDIT(5) = 1 edits the location and value of the MCHFR plus the MCHFR for each channel.	1
IEDIT(6)	IEDIT(6) = -1 to punch the power distribution at the end of each burnup step. IEDIT(6) = 0 for no power distribution punch. IEDIT(6) = 1 to punch the power distribution at the end of the last burnup step only.	0
A1	Nodal weighting factor for fast fluxes.	0.31
A2	Nodal weighting factor for thermal fluxes.	0.73
MAXI	Number of inner iterations per outer iteration.	3*
MAXO	Maximum number of outer (source) iterations per power iteration.	6*
EPSO	Source iteration convergence criterion. The source iteration is considered converged when the relative change in the fission source for each node is less than or equal to EPSO:	1.0E-6
	$\max_l \left( \frac{ S_l^I - S_l^{I-1} }{S_l^I} \right) \leq \text{EPSO}$	
EPSP	The power distribution iteration is considered con-	0.5E-2

<u>NAMelist</u> <u>Variable</u>	<u>Description</u>	<u>Default</u>
	verged when the relative change in power at each node is less than or equal to EPSP.	
ISOR	If ISOR = 1, successive overrelaxation will be used on the inner iterations. (Not recommended.)	0*
ALPHA	Overrelaxation factor used for outer iterations.	1.65*
IXE	If IXE = 0 no equilibrium xenon correction will be made.	1
ITSETS	Number of cross section table sets to be read.	1
IEXPTS	Number of exposure interpolating points used in the table sets.	1
IVDPTS	Number of void fraction interpolating points used in the table sets.	1
BUMASK	Exposure mask used by the table sets. BUMASK is an array of IEXPTS elements. Exposures must be in increasing order. (MWD/MTU)	0.
VMASK	Void fraction mask used by the table sets. VMASK is an array of IVDPTS elements. Void fractions must be in increasing order.	0.
ITSAS	Table set assignment format. If ITSAS = 1, table set #1 is assigned to all nodes. If ITSAS = 2, a one to one correspondence is used between fuel types and table sets (no axial variation). If ITSAS = 3, table set to fuel type overlays will be used.	2
IFTYPS	Number of fuel types used. (Required only if ITSAS = 3.)	1



<u>NAMELIST Variable</u>	<u>Description</u>	<u>Default</u>
TONNES	Core loading in Tonnes U. (Required only if burnup steps are input as hours.)	
PCTPWR	Core power in percent of rated power.	100.
MWTH	Rated reactor power (Mw-thermal).	
AVHFLX	Average heat flux at the clad surface at the rated reactor power. (Btu/hr ft <sup>2</sup> )	
PEAKF	Heat flux peaking factor. (Used in MCHFR calculation)	1.05
AREA	Active flow area per node. (ft <sup>2</sup> )	
HIN	Coolant inlet enthalpy. (Btu/lbm)	
HSAT	Saturation enthalpy at core average temperature and pressure. (Btu/lbm)	
HFG	Heat of vaporization. (Btu/lbm)	
VG	Specific volume of saturated steam. (ft <sup>3</sup> /lbm)	
VF	Specific volume of saturated water. (ft <sup>3</sup> /lbm)	
PSIA	Core average pressure. (psia)	
TFLOW	Total active incore coolant flow. (lbm/hr)	
AC1	Coefficients used in modified Armand correlation.	0.833
AC2	(See equation 4-11.)	0.167
IFLOW	If IFLOW = 0, the coolant flow in each channel will be assumed equal. If IFLOW = N, coefficients for a polynomial relating channel flow to the relative power of the channel will be read for N channel types.	0

Variables not used in a particular case need not be input. If  
 IVDPTS = 1, no thermal-hydraulic calculations will be done. A list of

variables required for fuel exposure, void fraction, MCHFR, and equilibrium xenon calculations is given below.

Fuel exposure: BUMASK, TONNES

Void fraction: VMASK, PCPTWR, MWTH, HIN, HSAT, HFG, VG, VF, TFLOW,  
AC1, AC2.

MCHFR: Same as for void fraction plus AVHFLX, PEAKF, AREA,  
PSIA.

Eq. Xenon: PCTPWR, MWTH.

### 3. Boundary Conditions.

See section 2.1.2 for a detailed description of boundary condition treatment.

$$ALB = A + B \cdot VOID(l)$$

ALB = 0.0 for zero flux,

= 1.0 for zero current,

= x.x for leakage specified by A and B.

If IGEOM = 1: Enter six pairs of constants, A and B, for the fast group and then six pairs for the thermal group. Boundary conditions must be entered, in order, for boundary faces one through six using the directional indexing convention depicted in figure 2.2.

If IGEOM = 2 (quarter core geometry): Enter two pairs of constants, A and B, for the bottom and then top of the core: first for the fast group, then for the thermal group. Next enter NX pairs of constants for boundary faces 1 through NX as shown in figure 2.4: first for the fast group, then for the thermal group.

4. Xe-135 + I-135 fission yields. (Ignore if IXE = 0)

IEXPTS yields are required for each table set corresponding to the exposures in the exposure mask. Begin a new record for each table set. For each table set enter the table set number followed by IEXPTS yield values.

## 5. Table set input.

The following sequence is repeated for each of ITSETS table sets.

Each table set begins with the following two cards:

- a) table set number.
- b) table set descriptor. FORMAT(20A4)

Cross section input is in the following order:

- a)  $D_1, \Sigma_{a1}, v\Sigma_{f1}, \kappa\Sigma_{f1}, \Sigma_r,$   
 $D_2, \Sigma_{a2}, v\Sigma_{f2}, \kappa\Sigma_{f2}$
- b)  $N^{Xe}, \sigma_{a2}^{Xe}, \Sigma_{f1}, \Sigma_{f2}$

Input in b above is included only if IXE = 1 and must begin on a separate record from list a.

The cross sections are input at the first void interpolating point at exposure points 1 through IEXPTS, then at the second void interpolating point at these exposures, and so on. For each table set, cross sections must be input at (IEXPTS) x (IVDPTS) state points.

Units for D are cm.;  $\Sigma_a, v\Sigma_f, \Sigma_r, \Sigma_f$  are  $\text{cm}^{-1}$ ;  $\kappa\Sigma_f$  is in Mev/cm;  $N^{Xe}$  is atoms/barn-cm;  $\sigma_{a2}^{Xe}$  is in barns.

$N^{Xe}$  and  $\sigma_{a2}^{Xe}$  must be consistent with  $\Sigma_{a2}$ .

## 6. Fuel type overlay. (Ignore if ITSAS = 1)

Enter NP integer fuel types corresponding to each planar coordinate in 2-D array format.

7. Table set to fuel type overlays. (Ignore if ITSAS  $\neq$  3)

IFTYPS overlays are required. Each overlay description begins on a new record. An overlay description consists of an integer fuel type number followed by NZ integer table set numbers corresponding to axial nodes 1 through NZ for that fuel type.

8. Control rod positions. (Ignore if ICNTRL = 0)

Rod positions are indicated as an integer, from 0 to NZ, giving the number of axial nodes that are rodged for each planar coordinate.

If NBANKS = 0: Enter NP control rod positions for each planar coordinate in 2-D array format.

If NBANKS  $\neq$  0: First define NBANKS control rod banks. Each bank definition must begin on a new record. A bank definition is input as a bank ID number, the number of planar coordinates assigned to the bank, and the I,J coordinates of each location. The last two coordinates must be followed by a slash(/). For example

4, 3, 7,1, 2,3, 8,11/

assigns coordinates (7,1), (2,3), and (8,11) to rod bank number 4.

After all banks are defined, each bank position is input. For each rod bank one record is required containing the bank ID followed by the axial position of the bank.

9. Control rod worths. (Ignore if ICNTRL = 0)

ICNTRL tables of control rod worths are required. For each table, the first record contains the number of the corresponding table set. This is followed by IVDPTS sets of rod worths. The first set contains IEXPTS worths for each successive exposure point in the exposure mask,

corresponding to the first point in the void fraction mask. The next set begins on a new record and contains the IEXPTS worths for the second void fraction, and so on.

10. Fuel exposures. (Ignore if IEXRD = 0)

If IEXRD = 1, enter the fuel exposure distribution in 3-D array format. Exposure units are MWD/MTU.

If IEXRD = -1, axial exposure distributions are input at selected planar coordinates by repeating the following sequence for each desired coordinate.

- a) I,J planar coordinate indices.
- b) Fuel exposures for axial nodes 1 through NZ in MWD/MTU.

If less than NP axial distributions are to be input, follow the last exposure distribution by one record containing -99/ . All nodes which are not assigned exposures are set at 0.0 MWD/MTU.

11. Power distribution guess. (Ignore if IPDRD  $\neq$  1)

Enter the zero'th power distribution iterant in 3-D array format.  
(Code will normalize)

12. Flow channel type overlay. (Ignore if IFLOW = 0)

Enter the flow channel type indices for each planar coordinate in 2-D array format. NP integers are required.

13. Channel flow versus power polynomials. (Ignore if IFLOW = 0)

Enter the coefficients of the flow versus power polynomials on one record for each of IFLOW channel types. Each record contains the channel type index followed by A, B, and C for that channel type. (See equation 4-3.)

#### 14. Burnup step input.

If IPTYPE = 2 (Haling solution) enter the exposure increment (cycle length) for which the solution is to be calculated in MWD/MTU. This completes the input for a Haling solution problem.

If IPTYPE = 1, and no burnup step calculations are desired, no further input is required. Only the power distribution at the initial input conditions will be calculated.

If IPTYPE = 1, and burnup step calculations are desired, the following input is needed.

##### Burnup step #1:

Enter the first burnup step increment in MWD/MTU or in hours. If burnup steps are input as hours, the values must be preceded by a negative sign and TONNES must have been included in the NAMELIST input.

For each additional step, the following input is required.

##### a) Burnup increment and update indicators:

BURN, NEWTH, NEWF, NEWCR

BURN = step length (MWD/MTU, or -hours)

NEWTH = 1 if thermal-hydraulic parameters are to be changed for this step. Otherwise set NEWTH = 0.

NEWF = N if the fuel loading is to be changed at N planar coordinates. This allows the fuel type overlay, channel type overlay, and fuel exposure array to be changed. Otherwise NEWF = 0.

NEWCR = 0 if control rod positions are to remain unchanged.

= N if NBANKS  $\neq$  0 and the positions of N rod banks are to be changed.

NEWCR = -1 if NBANKS = 0 and a new rod position array is to be read.

= N if NBANKS = 0 and the rod positions at N planar coordinates are to be changed.

b) Updated thermal-hydraulic data. (Ignore if NEWTH = 0)

PCTPWR, VG, VF, HIN, HSAT, HFG, PSIA, AC1, AC2, TFLOW, and PEAKF can be changed by repeating the NAMELIST input as described in #1. All variables listed above retain their previous values if unreferenced. Coefficients in the flow versus power polynomials can be changed by setting IFLOW to the number of polynomials to be changed. If IFLOW is reset to N, follow the NAMELIST input with N records containing new coefficients as in item 13.

c) Updated fuel loading. (Ignore if NEWF = 0)

The following sequence must be repeated NEWF times.

1) IFT, ICTYP, I, J

IFT = fuel type number

ICTYP = channel type number

I, J = planar coordinates

2) Exposures for nodes 1 through NZ (MWD/MTU)

d) Updated control rod positions. (Ignore if NEWCR = 0)

If NBANKS  $\neq$  0, NEWCR records must be input. Each record contains the rod bank ID number followed by the bank position.

If NBANKS = 0:

If NEWCR = -1, enter a new rod position array in 2-D array format.

If NEWCR = N, (N  $\neq$  0), NEWCR records must be input. Each re-

cord contains I, J coordinates followed by the rod position at those coordinates. The rod positions at all other coordinates remain unchanged.



Sample Input

EDWIN HATCH UNIT #2 100% POWER

7-11-77

&amp;INPUT

ICNTRL=3, IEDIT=0.0.0,-1.1,

MXI=2436.,

OX=15.240, DZ=15.875, NCOL=5\*13.12.3\*11.10.9.6.5,

IEAPTS=12, IVDPTS=3, IFTYPS=3, ITSETS=3, ITSAS=2,

EPSP=.1E-2.

BUMASK=0.,100.,500.,2000.,4000.,6000.,8000.,10000.,15000.,

20000.,25000.,30000.,

VMASK=0.,32.,64,

AVMFLX=145060., VG=.4297, VF=.0217, MSAT=547.8,

HFG=642.8, PSIA=1035., IFLOW=2, IFLOW=77.E6, AREA=.1098611,

MLN=526.9

&amp;END

.568 0. .568 0. .208 0. .208 0.

.568 0. .568 0. .568 0. .568 0. .568 0. .568 0. .568 0.

.568 0. .568 0. .568 0. .568 0. .568 0. .568 0. .568 0.

.208 0. .208 0. .208 0. .208 0. .208 0. .208 0. .208 0.

.208 0. .208 0. .208 0. .208 0. .208 0. .208 0. .208 0.

3 .0661 .0666 .0663 .0696 .0706 .0713 .0717 .0723 .0727 .073 .0731 .0732

1 .066 .0661 .0666 .0674 .0681 .0686 .0691 .0698 .0706 .0712 .0718 .0722

2 .066 .0661 .0665 .067 .0679 .068 .0684 .0691 .0698 .0705 .0711 .0716

3

OX5 BWR UNIT CELL .711 W/O

.142360+01 .600560-02 .257810-02 .202780+00 .233110-01

.378530+00 .253460-01 .233760-01 .201680+01

.0 .186950+07 .961820-03 .961970-02

.142140+01 .601600-02 .258100-02 .202910+00 .233610-01

.379020+00 .261250-01 .237730-01 .204630+01

.271850-09 .186330+07 .962030-03 .972880-02

.142430+01 .602080-02 .258780-02 .202800+00 .232840-01

.379400+00 .273130-01 .253660-01 .215090+01

.344410-09 .185600+07 .961180-03 .101890-01

.142950+01 .604420-02 .258520-02 .200940+00 .231410-01

.379630+00 .300000-01 .288290-01 .236630+01

.374970-09 .184240+07 .950310-03 .111140-01

.142890+01 .609290-02 .256920-02 .198460+00 .231320-01

.379600+00 .321540-01 .308020-01 .247680+01

.406290-09 .183250+07 .935170-03 .115350-01

.142670+01 .615030-02 .255410-02 .196520+00 .231590-01

.379560+00 .335440-01 .316850-01 .251720+01

.421560-09 .182640+07 .922700-03 .116450-01

.142420+01 .621310-02 .254110-02 .195040+00 .231900-01

.379540+00 .345270-01 .320780-01 .252770+01

.427400-09 .182200+07 .912570-03 .116300-01

.142200+01 .627920-02 .253060-02 .193920+00 .232130-01

.379560+00 .352790-01 .322360-01 .252550+01

.429130-09 .181860+07 .904450-03 .115680-01

.135680+01 .645450-02 .246180-02 .188720+00 .234050-01

.379760+00 .366210-01 .322800-01 .250730+01

.427600-09 .181170+07 .873560-03 .113830-01

.135340+01 .662800-02 .244650-02 .187720+00 .234370-01

.380090+00 .376740-01 .322380-01 .249500+01

.425050-09 .180560+07 .864030-03 .112600-01

.126940+01 .681590-02 .239560-02 .184650+00 .236790-01

.380530+00 .386360-01 .323130-01 .249740+01

.424900-09 .179940+07 .845490-03 .112280-01

.126850+01 .697420-02 .238950-02 .184640+00 .236770-01

.3*1000+00	.394940-01	.324310-01	.250640+01	
.425500-09	.179350+07	.442040-03	.112360-01	
.157880+01	.580170-02	.250190-02	.197200+00	.187200-01
.439*10+00	.240390-01	.230470-01	.198560+01	
.0	.183310+07	.925340-03	.948430-02	
.157640+01	.581300-02	.250550-02	.197390+00	.187650-01
.440510+00	.248320-01	.234800-01	.201790+01	
.277430-09	.182550+07	.935800-03	.960530-02	
.157990+01	.581860-02	.251600-02	.197530+00	.186890-01
.441120+00	.261050-01	.252270-01	.213390+01	
.352970-09	.181610+07	.936200-03	.101200-01	
.158600+01	.584660-02	.252400-02	.196430+00	.185500-01
.441660+00	.290650-01	.291260-01	.238250+01	
.388740-09	.179790+07	.929070-03	.112000-01	
.158610+01	.590100-02	.252030-02	.194850+00	.185220-01
.441860+00	.315150-01	.315820-01	.253020+01	
.427740-09	.178400+07	.918340-03	.117970-01	
.158450+01	.596290-02	.251600-02	.193700+00	.185230-01
.442000+00	.331560-01	.328920-01	.260340+01	
.450010-09	.177480+07	.909750-03	.120620-01	
.158250+01	.602920-02	.251220-02	.192890+00	.185290-01
.442140+00	.343520-01	.336550-01	.264240+01	
.461660-09	.176810+07	.902930-03	.121820-01	
.158050+01	.609830-02	.250950-02	.192330+00	.185330-01
.442280+00	.352840-01	.341270-01	.266430+01	
.468260-09	.176280+07	.897570-03	.122320-01	
.157620+01	.627440-02	.250700-02	.191780+00	.185250-01
.442710+00	.369160-01	.346770-01	.268430+01	
.472270-09	.175270+07	.889100-03	.122280-01	
.155740+01	.644330-02	.249030-02	.190590+00	.185300-01
.443260+00	.381850-01	.349930-01	.269850+01	
.476850-09	.174400+07	.878670-03	.122300-01	
.150990+01	.660780-02	.245950-02	.188780+00	.185890-01
.443900+00	.392900-01	.353010-01	.271820+01	
.480840-09	.173580+07	.866080-03	.122750-01	
.150860+01	.675930-02	.246140-02	.189290+00	.185530-01
.444560+00	.402660-01	.356110-01	.274100+01	
.484990-09	.172810+07	.865160-03	.123460-01	
.176780+01	.547630-02	.239720-02	.189610+00	.142250-01
.524660+00	.225620-01	.224910-01	.193550+01	
.0	.177840+07	.898910-03	.925530-02	
.176520+01	.548830-02	.240170-02	.189870+00	.142620-01
.525710+00	.233690-01	.229640-01	.197070+01	
.284660-09	.176860+07	.899690-03	.938980-02	
.176930+01	.549560-02	.241690-02	.190340+00	.141900-01
.526740+00	.247440-01	.248970-01	.210060+01	
.364920-09	.175570+07	.901730-03	.996980-02	
.177650+01	.553060-02	.243880-02	.190250+00	.140580-01
.527890+00	.280420-01	.293240-01	.238960+01	
.408010-09	.173030+07	.899460-03	.112400-01	
.177740+01	.559300-02	.245170-02	.189880+00	.140120-01
.528610+00	.308620-01	.323830-01	.258390+01	
.458110-09	.170990+07	.894650-03	.120560-01	
.177670+01	.566110-02	.246230-02	.189820+00	.139890-01
.529170+00	.328230-01	.342380-01	.269920+01	
.490670-09	.169570+07	.891390-03	.125210-01	
.177550+01	.573230-02	.247160-02	.189950+00	.139700-01
.529650+00	.342990-01	.354890-01	.277570+01	

.511030-09	.268500+07	.889220-03	.128190-01	
.177430+01	.580480-02	.248020-02	.190210+00	.139500-01
.530080+00	.354750-01	.383910-01	.283050+01	
.525110-09	.167630+07	.887890-03	.130250-01	
.177130+01	.598540-02	.249760-02	.191080+00	.139010-01
.531020+00	.375240-01	.376800-01	.290660+01	
.538070-09	.166040+07	.886300-03	.132860-01	
.176890+01	.615610-02	.251150-02	.192100+00	.138490-01
.531990+00	.391040-01	.385340-01	.296110+01	
.552980-09	.164730+07	.886520-03	.134740-01	
.176750+01	.631320-02	.252570-02	.193340+00	.137930-01
.533010+00	.404390-01	.392360-01	.300980+01	
.565110-09	.163550+07	.888590-03	.136530-01	
.176690+01	.645950-02	.253980-02	.194680+00	.137300-01
.534020+00	.416020-01	.398620-01	.305570+01	
.576320-09	.162480+07	.891620-03	.138310-01	
1				
8X= PWR UNIT CELL	1.83 W/G			
.144240+01	.652220-02	.378020-02	.299430+00	.211190-01
.389860+00	.397180-01	.547070-01	.461960+01	
.0	.174950+07	.145290-02	.225130-01	
.144510+01	.654020-02	.377960-02	.299460+00	.212350-01
.390550+00	.407040-01	.544740-01	.460600+01	
.590180-09	.174120+07	.145230-02	.223890-01	
.144680+01	.654840-02	.376270-02	.297840+00	.212900-01
.390770+00	.413960-01	.546730-01	.460960+01	
.731980-09	.173690+07	.144350-02	.223620-01	
.144850+01	.655500-02	.368910-02	.290920+00	.213540-01
.390310+00	.423990-01	.552900-01	.460990+01	
.736490-09	.173470+07	.140690-02	.222670-01	
.145170+01	.656640-02	.358790-02	.281780+00	.214610-01
.389450+00	.433010-01	.551620-01	.454880+01	
.736320-09	.173460+07	.135860-02	.218560-01	
.145530+01	.658170-02	.348900-02	.273070+00	.215790-01
.388560+00	.438420-01	.544150-01	.444870+01	
.728240-09	.173640+07	.131230-02	.212730-01	
.145910+01	.660080-02	.339420-02	.264860+00	.217000-01
.387730+00	.441120-01	.532920-01	.432560+01	
.714550-09	.173920+07	.126860-02	.205890-01	
.146300+01	.662240-02	.330410-02	.257150+00	.218150-01
.386970+00	.441820-01	.519310-01	.418930+01	
.697360-09	.174260+07	.122730-02	.198500-01	
.145650+01	.669070-02	.311160-02	.240930+00	.220800-01
.385460+00	.439210-01	.483100-01	.384860+01	
.663020-09	.175140+07	.113980-02	.180420-01	
.144490+01	.677540-02	.294750-02	.227480+00	.223240-01
.384260+00	.433330-01	.446040-01	.351980+01	
.614260-09	.176000+07	.106680-02	.163330-01	
.143410+01	.687650-02	.281510-02	.216910+00	.225480-01
.383440+00	.427640-01	.413030-01	.323640+01	
.568140-09	.176680+07	.100890-02	.148730-01	
.140970+01	.698970-02	.269660-02	.207790+00	.227560-01
.382930+00	.423690-01	.386620-01	.301460+01	
.529230-09	.177130+07	.959010-03	.137350-01	
.160080+01	.631480-02	.370700-02	.294120+00	.168190-01
.454860+00	.379110-01	.534530-01	.451180+01	
.0	.169290+07	.142710-02	.219970-01	
.160340+01	.633370-02	.370710-02	.294210+00	.169190-01

.455810+00	.388870-01	.532260-01	.449830+01	
.605680-09	.168290+07	.142680-02	.218730-01	
.160470+01	.634360-02	.369300-02	.292790+00	.169630-01
.456100+00	.396450-01	.535520-01	.451110+01	
.751980-09	.167740+07	.141890-02	.218900-01	
.160660+01	.635540-02	.362890-02	.286550+00	.170100-01
.455690+00	.409180-01	.546210-01	.454560+01	
.760620-09	.167300+07	.138570-02	.219570-01	
.160970+01	.637380-02	.353990-02	.278310+00	.170930-01
.454730+00	.421520-01	.550220-01	.452530+01	
.760520-09	.167080+07	.134180-02	.217400-01	
.161300+01	.639550-02	.345320-02	.270500+00	.171830-01
.453730+00	.429980-01	.547730-01	.446390+01	
.764310-09	.167090+07	.130000-02	.213400-01	
.161680+01	.642040-02	.337030-02	.263170+00	.172730-01
.452820+00	.435480-01	.541290-01	.437850+01	
.755900-09	.167230+07	.126060-02	.208350-01	
.162000+01	.644730-02	.329160-02	.256310+00	.173570-01
.451980+00	.438790-01	.532270-01	.427800+01	
.743590-09	.167460+07	.122360-02	.202690-01	
.161630+01	.652550-02	.312210-02	.241800+00	.175430-01
.450320+00	.441410-01	.505590-01	.401200+01	
.710440-09	.168120+07	.114450-02	.188190-01	
.160570+01	.661560-02	.297600-02	.229670+00	.177090-01
.449020+00	.439820-01	.476370-01	.374460+01	
.676290-09	.168830+07	.107820-02	.174050-01	
.159600+01	.671690-02	.285570-02	.219970+00	.178570-01
.448090+00	.437280-01	.449110-01	.350610+01	
.636880-09	.169420+07	.102460-02	.161590-01	
.158780+01	.682830-02	.276140-02	.212590+00	.179720-01
.447500+00	.435350-01	.426300-01	.331190+01	
.602530-09	.169820+07	.983320-03	.151510-01	
.179680+01	.599010-02	.360570-02	.286860+00	.126210-01
.545200+00	.356480-01	.514100-01	.433840+01	
.6	.161100+07	.139130-02	.211570-01	
.179890+01	.601030-02	.360680-02	.287030+00	.127050-01
.546540+00	.365990-01	.511840-01	.432440+01	
.625470-09	.159880+07	.139140-02	.210300-01	
.120010+01	.602200-02	.359620-02	.285870+00	.127390-01
.547120+00	.374370-01	.516630-01	.434840+01	
.779780-09	.159120+07	.138490-02	.211000-01	
.180190+01	.604020-02	.354430-02	.280500+00	.127670-01
.546750+00	.390490-01	.532880-01	.442510+01	
.794420-09	.158320+07	.135600-02	.213660-01	
.180480+01	.606770-02	.347120-02	.273420+00	.128260-01
.545730+00	.407000-01	.543420-01	.445550+01	
.809240-09	.157740+07	.131780-02	.213900-01	
.180800+01	.609830-02	.340070-02	.266820+00	.128900-01
.544710+00	.419270-01	.547190-01	.444310+01	
.815640-09	.157450+07	.128190-02	.212230-01	
.181100+01	.612990-02	.333350-02	.260650+00	.129460-01
.543780+00	.428310-01	.546820-01	.440520+01	
.814940-09	.157340+07	.124830-02	.209460-01	
.181410+01	.616400-02	.327020-02	.254940+00	.129990-01
.542950+00	.434950-01	.543700-01	.435120+01	
.809860-09	.157330+07	.121690-02	.206020-01	
.181050+01	.625530-02	.313250-02	.242770+00	.131090-01
.541310+00	.444420-01	.529560-01	.418390+01	

.793290-09	.157550+07	.114940-02	.196260-01	
.120150+01	.635340-02	.301280-02	.232590+00	.132010-01
.540030+00	.448770-01	.511280-01	.400230+01	
.766290-09	.157890+07	.109270-02	.186230-01	
.179350+01	.645690-02	.291270-02	.224330+00	.132790-01
.539130+00	.451010-01	.492550-01	.383250+01	
.737870-09	.158200+07	.104640-02	.177060-01	
.178880+01	.656600-02	.283220-02	.217910+00	.133360-01
.538560+00	.452610-01	.476530-01	.368820+01	
.712030-09	.158400+07	.100980-02	.169350-01	

2  
BX- BWR UNIT CELL

2.33 W/U				
.136180+01	.677980-02	.434370-02	.344230+00	.204010-01
.393980+00	.453840-01	.670260-01	.564240+01	
.0	.170400+07	.167810-02	.275830-01	
.142100+01	.680170-02	.430600-02	.341960+00	.206360-01
.394710+00	.464490-01	.666150-01	.561620+01	
.705870-09	.169500+07	.166650-02	.273890-01	.207360-01
.144070+01	.680820-02	.426920-02	.338930+00	
.394940+00	.471050-01	.665800-01	.560320+01	
.875610-09	.169080+07	.165080-02	.272800-01	
.144260+01	.680960-02	.418460-02	.331240+00	.208040-01
.394420+00	.478340-01	.667190-01	.556930+01	
.876180-09	.169020+07	.161050-02	.270260-01	
.144540+01	.681350-02	.406930-02	.321040+00	.209120-01
.393490+00	.484790-01	.662030-01	.547900+01	
.871450-09	.169170+07	.155690-02	.264790-01	
.144860+01	.681990-02	.395490-02	.311100+00	.210290-01
.392510+00	.486460-01	.651710-01	.535590+01	
.860130-09	.169480+07	.150460-02	.257830-01	
.145190+01	.682890-02	.384320-02	.301530+00	.211500-01
.391580+00	.489840-01	.637990-01	.521180+01	
.844150-09	.169870+07	.145410-02	.249940-01	
.145530+01	.683940-02	.373480-02	.292320+00	.212680-01
.390660+00	.489380-01	.621900-01	.505330+01	
.824930-09	.170340+07	.140540-02	.241450-01	
.146410+01	.687580-02	.349260-02	.271970+00	.215460-01
.388770+00	.483680-01	.578360-01	.464690+01	
.785850-09	.171530+07	.129740-02	.220040-01	
.146180+01	.692440-02	.327510-02	.254040+00	.218140-01
.387160+00	.473630-01	.531670-01	.423260+01	
.728560-09	.172790+07	.120190-02	.198620-01	
.145010+01	.698750-02	.308520-02	.238680+00	.220710-01
.385840+00	.462460-01	.486400-01	.384280+01	
.670140-09	.173960+07	.111980-02	.178670-01	
.143900+01	.706980-02	.292650-02	.226090+00	.223120-01
.384820+00	.452140-01	.446020-01	.350220+01	
.615520-09	.174940+07	.105200-02	.161350-01	
.157760+01	.657080-02	.422820-02	.336190+00	.162960-01
.480210+00	.433240-01	.652770-01	.549360+01	
.0	.164060+07	.163900-02	.268630-01	
.159870+01	.658980-02	.421280-02	.335200+00	.164270-01
.461210+00	.443670-01	.648620-01	.546660+01	
.725200-09	.163000+07	.163340-02	.266660-01	
.160010+01	.659980-02	.419610-02	.333630+00	.164790-01
.461560+00	.450810-01	.649420-01	.546200+01	
.900360-09	.162460+07	.162470-02	.265960-01	
.160260+01	.660660-02	.412100-02	.326630+00	.165320-01

.461010+00	.460730-01	.655240-01	.546140+01	
.905110-09	.162220+07	.158780-02	.264990-01	
.160480+01	.661770-02	.401800-02	.317320+00	.166180-01
.459730+00	.470490-01	.655340-01	.541160+01	
.905790-09	.162200+07	.153870-02	.261440-01	
.160790+01	.663150-02	.391610-02	.308310+00	.167120-01
.458790+00	.477250-01	.650040-01	.532750+01	
.899740-09	.162370+07	.149100-02	.256340-01	
.161100+01	.664610-02	.381650-02	.299630+00	.168020-01
.457700+00	.481510-01	.641180-01	.522120+01	
.688500-09	.162650+07	.144490-02	.250260-01	
.161420+01	.666260-02	.372010-02	.291320+00	.168910-01
.456690+00	.483770-01	.629830-01	.510000+01	
.873760-09	.163020+07	.140070-02	.243550-01	
.162220+01	.671100-02	.350360-02	.272900+00	.170940-01
.454500+00	.483810-01	.596700-01	.477550+01	
.841780-09	.164030+07	.130220-02	.226080-01	
.162230+01	.676840-02	.330820-02	.256620+00	.172850-01
.452750+00	.478820-01	.559340-01	.443490+01	
.793320-09	.165150+07	.121490-02	.208210-01	
.161170+01	.683590-02	.313570-02	.242540+00	.174650-01
.451260+00	.471830-01	.521870-01	.410660+01	
.742610-09	.166220+07	.113930-02	.191230-01	
.160170+01	.691770-02	.298890-02	.230790+00	.176280-01
.450080+00	.464660-01	.487350-01	.381210+01	
.694160-09	.167160+07	.107580-02	.176120-01	
.179390+01	.624090-02	.410570-02	.327380+00	.122000-01
.552250+00	.406750-01	.624690-01	.525670+01	
.0	.155040+07	.159540-02	.257070-01	
.179610+01	.626170-02	.410770-02	.327650+00	.122820-01
.553640+00	.416780-01	.620420-01	.522810+01	
.750330-09	.153750+07	.154600-02	.255030-01	
.179740+01	.627370-02	.409460-02	.326350+00	.123220-01
.554210+00	.424610-01	.622610-01	.523350+01	
.935220-09	.153030+07	.158860-02	.254810-01	
.179920+01	.628780-02	.403170-02	.320230+00	.123590-01
.553700+00	.437800-01	.633850-01	.527410+01	
.945460-09	.152500+07	.155610-02	.255780-01	
.180210+01	.630880-02	.394460-02	.312110+00	.124260-01
.552490+00	.451700-01	.640430-01	.527450+01	
.954050-09	.152180+07	.151280-02	.254600-01	
.180500+01	.633030-02	.385850-02	.304260+00	.124900-01
.551230+00	.462300-01	.641340-01	.523890+01	
.955720-09	.152090+07	.147080-02	.251810-01	
.180780+01	.635310-02	.377500-02	.296780+00	.125530-01
.550050+00	.470180-01	.638580-01	.518070+01	
.951560-09	.152150+07	.143060-02	.248040-01	
.181070+01	.637710-02	.369450-02	.289650+00	.126130-01
.548970+00	.475870-01	.633240-01	.510680+01	
.943430-09	.152310+07	.139220-02	.243600-01	
.181780+01	.644130-02	.351260-02	.273800+00	.127440-01
.546740+00	.483260-01	.613550-01	.488840+01	
.921700-09	.152920+07	.130640-02	.231230-01	
.181730+01	.651050-02	.334800-02	.259810+00	.128620-01
.544870+00	.485000-01	.588690-01	.464660+01	
.886370-09	.153690+07	.123050-02	.216120-01	
.180840+01	.658500-02	.320150-02	.247640+00	.129680-01
.543320+00	.483910-01	.562290-01	.440560+01	

[illegible]

TIME STEP # 1      << EDWIN HATCH UNIT #2      100% POWER      7-11-77      >>      PAGE 1

NODE WIDTH = 15.240 CM.  
NODE HEIGHT = 15.475 CM.

NODAL WEIGHTING FACTORS: A1 = 0.3100  
A2 = 0.7300

\*\*\*\*\* BOUNDARY CONDITIONS \*\*\*\*\*

ALB = A + B\*VOID(I)

ALB = 1. FOR ZERO CURRENT

0. FOR ZERO FLUX

XX. FOR LEAKAGE SPECIFIED BY A AND B

--- FAST ---      = THERMAL =

	A	B	A	B
	*****	*****	*****	*****
ROTTOM	0.57	0.0	0.21	0.0
TOP	0.57	0.0	0.21	0.0

DIR. CORE EDGE

ROW NUMBER

1	0.57	0.0	0.21	0.0
2	0.57	0.0	0.21	0.0
3	0.57	0.0	0.21	0.0
4	0.57	0.0	0.21	0.0
5	0.57	0.0	0.21	0.0
6	0.57	0.0	0.21	0.0
7	0.57	0.0	0.21	0.0
8	0.57	0.0	0.21	0.0
9	0.57	0.0	0.21	0.0
10	0.57	0.0	0.21	0.0
11	0.57	0.0	0.21	0.0
12	0.57	0.0	0.21	0.0
13	0.57	0.0	0.21	0.0

\*\*\*\*\* CONVERGENCE PARAMETERS \*\*\*\*\*

OUTER ITERATION WILL BE CONVERGED TO 0.10E-05  
POWER ITERATION WILL BE CONVERGED TO 0.10E-02

MAXIMUM NUMBER OF INNER ITERATIONS = 3

MAXIMUM NUMBER OF OUTER ITERATIONS = 6

MAXIMUM NUMBER OF POWER ITERATIONS = 30



TIME STEP # 1 << EDWIN MATCH UNIT #2 100% POWER 7-11-77 >> PAGE 2

3 CROSS SECTION INTERPOLATING TABLES WILL BE READ AT 12 EXPOSURE AND 3 VOID INTERPOLATING POINTS

• EXPOSURE MASK •

TABLE EXPOSURE  
INDEX (MWD/MT)  
\*\*\*\*\*

1	0.
2	100.
3	500.
4	2000.
5	4000.
6	6000.
7	8000.
8	10000.
9	15000.
10	20000.
11	25000.
12	30000.

• VOID MASK •

TABLE  
INDEX VOID  
\*\*\*\*\*

1	0.0
2	0.320
3	0.640

\*\*\*\*\* CROSS SECTION INTERPOLATING TABLE \*\*\*\*\*

TABLE SET 3      8X8 BWR UNIT CELL      .711 M20

***** F I S I O N *****						***** T H E O R E T I C A L *****			
EXPOSURE (MWD/KT)	0	ABSORPTION	FISSION	KAPPA- FISSION	DOWN SCATTERING	0	ABSORPTION	FISSION	KAPPA- FISSION
VOID = 0.0									
0.	0.1424E+01	0.6006E-02	0.2578E-02	0.2028E+00	0.2331E-01	0.3785E+00	0.2535E-01	0.2338E-01	0.2017E+01
100.	0.1421E+01	0.6016E-02	0.2581E-02	0.2029E+00	0.2336E-01	0.3790E+00	0.2612E-01	0.2377E-01	0.2046E+01
500.	0.1424E+01	0.6021E-02	0.2588E-02	0.2028E+00	0.2328E-01	0.3794E+00	0.2731E-01	0.2537E-01	0.2151E+01
2000.	0.1429E+01	0.6044E-02	0.2585E-02	0.2009E+00	0.2314E-01	0.3796E+00	0.3000E-01	0.2883E-01	0.2366E+01
4000.	0.1429E+01	0.6093E-02	0.2569E-02	0.1985E+00	0.2313E-01	0.3796E+00	0.3215E-01	0.3080E-01	0.2477E+01
6000.	0.1427E+01	0.6150E-02	0.2554E-02	0.1965E+00	0.2316E-01	0.3796E+00	0.3354E-01	0.3168E-01	0.2517E+01
8000.	0.1424E+01	0.6213E-02	0.2541E-02	0.1950E+00	0.2319E-01	0.3795E+00	0.3453E-01	0.3208E-01	0.2528E+01
10000.	0.1422E+01	0.6279E-02	0.2531E-02	0.1939E+00	0.2321E-01	0.3796E+00	0.3528E-01	0.3224E-01	0.2526E+01
15000.	0.1357E+01	0.6455E-02	0.2462E-02	0.1887E+00	0.2341E-01	0.3798E+00	0.3662E-01	0.3228E-01	0.2507E+01
20000.	0.1353E+01	0.6628E-02	0.2446E-02	0.1877E+00	0.2344E-01	0.3801E+00	0.3767E-01	0.3224E-01	0.2495E+01
25000.	0.1269E+01	0.6816E-02	0.2396E-02	0.1847E+00	0.2368E-01	0.3805E+00	0.3864E-01	0.3231E-01	0.2497E+01
30000.	0.1269E+01	0.6974E-02	0.2389E-02	0.1846E+00	0.2368E-01	0.3810E+00	0.3949E-01	0.3243E-01	0.2506E+01
VOID = 0.32									
0.	0.1579E+01	0.5802E-02	0.2502E-02	0.1972E+00	0.1872E-01	0.4398E+00	0.2404E-01	0.2305E-01	0.1986E+01
100.	0.1576E+01	0.5813E-02	0.2506E-02	0.1974E+00	0.1876E-01	0.4405E+00	0.2483E-01	0.2348E-01	0.2018E+01
500.	0.1580E+01	0.5819E-02	0.2516E-02	0.1975E+00	0.1869E-01	0.4411E+00	0.2611E-01	0.2523E-01	0.2134E+01
2000.	0.1586E+01	0.5847E-02	0.2524E-02	0.1964E+00	0.1855E-01	0.4417E+00	0.2907E-01	0.2913E-01	0.2382E+01
4000.	0.1586E+01	0.5901E-02	0.2520E-02	0.1949E+00	0.1852E-01	0.4419E+00	0.3151E-01	0.3150E-01	0.2530E+01
6000.	0.1585E+01	0.5963E-02	0.2516E-02	0.1937E+00	0.1852E-01	0.4420E+00	0.3316E-01	0.3289E-01	0.2603E+01
8000.	0.1583E+01	0.6029E-02	0.2512E-02	0.1929E+00	0.1853E-01	0.4421E+00	0.3435E-01	0.3365E-01	0.2642E+01
10000.	0.1580E+01	0.6098E-02	0.2509E-02	0.1923E+00	0.1853E-01	0.4423E+00	0.3528E-01	0.3413E-01	0.2664E+01
15000.	0.1576E+01	0.6274E-02	0.2507E-02	0.1918E+00	0.1853E-01	0.4427E+00	0.3692E-01	0.3468E-01	0.2684E+01
20000.	0.1557E+01	0.6443E-02	0.2490E-02	0.1906E+00	0.1853E-01	0.4433E+00	0.3819E-01	0.3499E-01	0.2698E+01
25000.	0.1510E+01	0.6608E-02	0.2459E-02	0.1888E+00	0.1859E-01	0.4439E+00	0.3929E-01	0.3530E-01	0.2710E+01
30000.	0.1509E+01	0.6759E-02	0.2461E-02	0.1893E+00	0.1855E-01	0.4446E+00	0.4027E-01	0.3561E-01	0.2741E+01
VOID = 0.64									
0.	0.1768E+01	0.5476E-02	0.2397E-02	0.1896E+00	0.1422E-01	0.5247E+00	0.2256E-01	0.2249E-01	0.1936E+01
100.	0.1765E+01	0.5488E-02	0.2402E-02	0.1899E+00	0.1426E-01	0.5257E+00	0.2337E-01	0.2296E-01	0.1971E+01
500.	0.1769E+01	0.5496E-02	0.2417E-02	0.1903E+00	0.1419E-01	0.5267E+00	0.2474E-01	0.2490E-01	0.2101E+01
2000.	0.1776E+01	0.5531E-02	0.2439E-02	0.1902E+00	0.1406E-01	0.5279E+00	0.2804E-01	0.2932E-01	0.2390E+01
4000.	0.1777E+01	0.5593E-02	0.2452E-02	0.1899E+00	0.1401E-01	0.5286E+00	0.3086E-01	0.3238E-01	0.2584E+01
6000.	0.1777E+01	0.5661E-02	0.2462E-02	0.1894E+00	0.1399E-01	0.5292E+00	0.3282E-01	0.3424E-01	0.2699E+01
8000.	0.1776E+01	0.5732E-02	0.2472E-02	0.1899E+00	0.1397E-01	0.5296E+00	0.3430E-01	0.3549E-01	0.2776E+01
10000.	0.1774E+01	0.5805E-02	0.2480E-02	0.1902E+00	0.1395E-01	0.5301E+00	0.3548E-01	0.3639E-01	0.2839E+01
15000.	0.1771E+01	0.5945E-02	0.2494E-02	0.1911E+00	0.1390E-01	0.5310E+00	0.3752E-01	0.3768E-01	0.2907E+01
20000.	0.1769E+01	0.6156E-02	0.2511E-02	0.1921E+00	0.1385E-01	0.5320E+00	0.3910E-01	0.3853E-01	0.2961E+01
25000.	0.1767E+01	0.6313E-02	0.2526E-02	0.1933E+00	0.1379E-01	0.5330E+00	0.4044E-01	0.3924E-01	0.3010E+01
30000.	0.1767E+01	0.6460E-02	0.2540E-02	0.1947E+00	0.1373E-01	0.5340E+00	0.4160E-01	0.3986E-01	0.3056E+01

## CROSS SECTION INTERPOLATING TABLE

TABLE SET 3 8X8 BWR UNIT CELL .711 W/D

EXPOSURE (MWD/MT)	THERMAL		FAST		THERMAL	
	ABSORPTION XE REMOVED		FISSION		ABSORPTION MICRO	
*****	*****	*****	*****	*****	*****	*****
VOID = 0.0						
0.	0.2535E-01	0.9618E-03	0.9620E-02	0.9620E-02	1869500.	
100.	0.2562E-01	0.9620E-03	0.9729E-02	0.9729E-02	1863300.	
500.	0.2667E-01	0.9612E-03	0.1019E-01	0.1019E-01	1856000.	
2000.	0.2931E-01	0.9503E-03	0.1111E-01	0.1111E-01	1842600.	
4000.	0.3141E-01	0.9352E-03	0.1154E-01	0.1154E-01	1832500.	
6000.	0.3277E-01	0.9227E-03	0.1165E-01	0.1165E-01	1826400.	
8000.	0.3375E-01	0.9126E-03	0.1163E-01	0.1163E-01	1822000.	
10000.	0.3450E-01	0.9045E-03	0.1157E-01	0.1157E-01	1818600.	
15000.	0.3505E-01	0.8736E-03	0.1130E-01	0.1130E-01	1811700.	
20000.	0.3691E-01	0.8640E-03	0.1126E-01	0.1126E-01	1805600.	
25000.	0.3787E-01	0.8455E-03	0.1123E-01	0.1123E-01	1799400.	
30000.	0.3873E-01	0.8420E-03	0.1124E-01	0.1124E-01	1793500.	
VOID = 0.32						
0.	0.2404E-01	0.9353E-03	0.9484E-02	0.9484E-02	1833100.	
100.	0.2433E-01	0.9350E-03	0.9605E-02	0.9605E-02	1825500.	
500.	0.2546E-01	0.9362E-03	0.1012E-01	0.1012E-01	1816100.	
2000.	0.2837E-01	0.9291E-03	0.1120E-01	0.1120E-01	1797900.	
4000.	0.3075E-01	0.9183E-03	0.1180E-01	0.1180E-01	1784000.	
6000.	0.3236E-01	0.9098E-03	0.1206E-01	0.1206E-01	1774800.	
8000.	0.3354E-01	0.9029E-03	0.1218E-01	0.1218E-01	1768100.	
10000.	0.3446E-01	0.8976E-03	0.1223E-01	0.1223E-01	1762800.	
15000.	0.3609E-01	0.8891E-03	0.1223E-01	0.1223E-01	1752700.	
20000.	0.3735E-01	0.8747E-03	0.1223E-01	0.1223E-01	1744000.	
25000.	0.3846E-01	0.8661E-03	0.1228E-01	0.1228E-01	1735800.	
30000.	0.3943E-01	0.8652E-03	0.1235E-01	0.1235E-01	1728100.	
VOID = 0.64						
0.	0.2256E-01	0.8949E-03	0.9255E-02	0.9255E-02	1778400.	
100.	0.2287E-01	0.8997E-03	0.9390E-02	0.9390E-02	1768600.	
500.	0.2410E-01	0.9017E-03	0.9970E-02	0.9970E-02	1755700.	
2000.	0.2734E-01	0.8995E-03	0.1129E-01	0.1129E-01	1730300.	
4000.	0.3008E-01	0.8946E-03	0.1206E-01	0.1206E-01	1709900.	
6000.	0.3199E-01	0.8914E-03	0.1252E-01	0.1252E-01	1695700.	
8000.	0.3293E-01	0.8892E-03	0.1282E-01	0.1282E-01	1685000.	
10000.	0.3459E-01	0.8879E-03	0.1303E-01	0.1303E-01	1676300.	
15000.	0.3663E-01	0.8863E-03	0.1329E-01	0.1329E-01	1660400.	
20000.	0.3819E-01	0.8865E-03	0.1347E-01	0.1347E-01	1647300.	
25000.	0.3951E-01	0.8846E-03	0.1365E-01	0.1365E-01	1635500.	
30000.	0.4067E-01	0.8916E-03	0.1383E-01	0.1383E-01	1624800.	

TIME STEP # 1 << EDWIN HATCH UNIT #2 100% POWER 7-11-77 >> PAGE 5

\*\*\*\*\* CROSS SECTION INTERPOLATING TABLE \*\*\*\*\*

TABLE SET 1 0X0 HWR UNIT CELL 1.03 M/0

***** F A S I *****									
***** T H E R M A I *****									
EXPOSURE (MW/D/MT)	D	ABSORPTION	NU- FISSION	KAPPA- FISSION	DOWN SCATTERING	D	ABSORPTION	NU- FISSION	KAPPA- FISSION
VOID = 0.0									
0.	0.1442E+01	0.6522E-02	0.3780E-02	0.2994E+00	0.2112E-01	0.3899E+00	0.3972E-01	0.5471E-01	0.4620E+01
100.	0.1445E+01	0.6540E-02	0.3780E-02	0.2995E+00	0.2124E-01	0.3906E+00	0.4070E-01	0.5447E-01	0.4606E+01
500.	0.1447E+01	0.6548E-02	0.3763E-02	0.2978E+00	0.2129E-01	0.3908E+00	0.4140E-01	0.5467E-01	0.4610E+01
2000.	0.1448E+01	0.6555E-02	0.3689E-02	0.2909E+00	0.2135E-01	0.3903E+00	0.4240E-01	0.5529E-01	0.4610E+01
4000.	0.1452E+01	0.6566E-02	0.3588E-02	0.2818E+00	0.2146E-01	0.3895E+00	0.4330E-01	0.5516E-01	0.4549E+01
6000.	0.1455E+01	0.6582E-02	0.3489E-02	0.2731E+00	0.2158E-01	0.3886E+00	0.4384E-01	0.5441E-01	0.4449E+01
8000.	0.1459E+01	0.6601E-02	0.3394E-02	0.2649E+00	0.2170E-01	0.3877E+00	0.4411E-01	0.5329E-01	0.4326E+01
10000.	0.1463E+01	0.6622E-02	0.3304E-02	0.2571E+00	0.2181E-01	0.3870E+00	0.4418E-01	0.5193E-01	0.4189E+01
15000.	0.1457E+01	0.6691E-02	0.3112E-02	0.2409E+00	0.2200E-01	0.3855E+00	0.4392E-01	0.4831E-01	0.3849E+01
20000.	0.1445E+01	0.6715E-02	0.2947E-02	0.2275E+00	0.2232E-01	0.3843E+00	0.4333E-01	0.4660E-01	0.3520E+01
25000.	0.1434E+01	0.6876E-02	0.2815E-02	0.2164E+00	0.2255E-01	0.3834E+00	0.4276E-01	0.4130E-01	0.3236E+01
30000.	0.1410E+01	0.6990E-02	0.2697E-02	0.2078E+00	0.2276E-01	0.3829E+00	0.4217E-01	0.3866E-01	0.3015E+01
VOID = 0.32									
0.	0.1601E+01	0.6315E-02	0.3707E-02	0.2941E+00	0.1682E-01	0.4549E+00	0.3791E-01	0.5345E-01	0.4512E+01
100.	0.1603E+01	0.6334E-02	0.3707E-02	0.2942E+00	0.1692E-01	0.4554E+00	0.3889E-01	0.5323E-01	0.4498E+01
500.	0.1605E+01	0.6344E-02	0.3693E-02	0.2928E+00	0.1696E-01	0.4562E+00	0.3965E-01	0.5355E-01	0.4511E+01
2000.	0.1607E+01	0.6355E-02	0.3629E-02	0.2865E+00	0.1701E-01	0.4557E+00	0.4092E-01	0.5462E-01	0.4546E+01
4000.	0.1610E+01	0.6374E-02	0.3540E-02	0.2783E+00	0.1709E-01	0.4547E+00	0.4215E-01	0.5502E-01	0.4525E+01
6000.	0.1613E+01	0.6396E-02	0.3453E-02	0.2705E+00	0.1718E-01	0.4537E+00	0.4300E-01	0.5477E-01	0.4464E+01
8000.	0.1617E+01	0.6420E-02	0.3370E-02	0.2632E+00	0.1727E-01	0.4528E+00	0.4355E-01	0.5413E-01	0.4378E+01
10000.	0.1620E+01	0.6447E-02	0.3292E-02	0.2563E+00	0.1736E-01	0.4520E+00	0.4388E-01	0.5323E-01	0.4278E+01
15000.	0.1616E+01	0.6526E-02	0.3122E-02	0.2418E+00	0.1754E-01	0.4503E+00	0.4414E-01	0.5056E-01	0.4012E+01
20000.	0.1606E+01	0.6616E-02	0.2976E-02	0.2297E+00	0.1771E-01	0.4490E+00	0.4398E-01	0.4764E-01	0.3745E+01
25000.	0.1596E+01	0.6717E-02	0.2856E-02	0.2200E+00	0.1786E-01	0.4481E+00	0.4373E-01	0.4491E-01	0.3508E+01
30000.	0.1588E+01	0.6828E-02	0.2761E-02	0.2126E+00	0.1797E-01	0.4475E+00	0.4354E-01	0.4263E-01	0.3312E+01
VOID = 0.64									
0.	0.1797E+01	0.5990E-02	0.3606E-02	0.2864E+00	0.1262E-01	0.5452E+00	0.3565E-01	0.5141E-01	0.4338E+01
100.	0.1799E+01	0.6018E-02	0.3607E-02	0.2870E+00	0.1270E-01	0.5465E+00	0.3660E-01	0.5118E-01	0.4324E+01
500.	0.1800E+01	0.6022E-02	0.3596E-02	0.2859E+00	0.1274E-01	0.5471E+00	0.3744E-01	0.5166E-01	0.4348E+01
2000.	0.1802E+01	0.6040E-02	0.3544E-02	0.2805E+00	0.1277E-01	0.5468E+00	0.3905E-01	0.5329E-01	0.4425E+01
4000.	0.1805E+01	0.6068E-02	0.3471E-02	0.2734E+00	0.1283E-01	0.5457E+00	0.4070E-01	0.5434E-01	0.4455E+01
6000.	0.1808E+01	0.6098E-02	0.3401E-02	0.2668E+00	0.1289E-01	0.5447E+00	0.4193E-01	0.5472E-01	0.4443E+01
8000.	0.1811E+01	0.6130E-02	0.3333E-02	0.2606E+00	0.1295E-01	0.5438E+00	0.4283E-01	0.5468E-01	0.4405E+01
10000.	0.1814E+01	0.6164E-02	0.3270E-02	0.2549E+00	0.1300E-01	0.5429E+00	0.4359E-01	0.5437E-01	0.4351E+01
15000.	0.1811E+01	0.6255E-02	0.3132E-02	0.2428E+00	0.1311E-01	0.5413E+00	0.4444E-01	0.5296E-01	0.4184E+01
20000.	0.1802E+01	0.6353E-02	0.3013E-02	0.2326E+00	0.1320E-01	0.5400E+00	0.4488E-01	0.5113E-01	0.4002E+01
25000.	0.1793E+01	0.6457E-02	0.2913E-02	0.2243E+00	0.1328E-01	0.5391E+00	0.4510E-01	0.4928E-01	0.3833E+01
30000.	0.1787E+01	0.6566E-02	0.2832E-02	0.2179E+00	0.1334E-01	0.5386E+00	0.4526E-01	0.4765E-01	0.3688E+01

TIME STEP # 1 << EDWIN HATCH UNIT #2 100% POWER 7-11-77 >> PAGE 6

CROSS SECTION INTERPOLATING TABLE

TABLE SET 1 BXB BWR UNIT CELL 1.43 W/O

EXPOSURE (MWDZMT)	ABSORPTION XE REMOVED	FAST FISSION	THERMAL FISSION	XL-135 MICRO ABSORPTION
VOID = 0.0				
0.	0.3972E-01	0.1453E-02	0.2251E-01	1749500.
100.	0.3968E-01	0.1452E-02	0.2239E-01	1741200.
500.	0.4012E-01	0.1444E-02	0.2236E-01	1736900.
2000.	0.4112E-01	0.1407E-02	0.2227E-01	1734700.
4000.	0.4202E-01	0.1359E-02	0.2186E-01	1734600.
6000.	0.4258E-01	0.1312E-02	0.2127E-01	1736400.
8000.	0.4287E-01	0.1269E-02	0.2059E-01	1739200.
10000.	0.4297E-01	0.1227E-02	0.1985E-01	1742600.
15000.	0.4276E-01	0.1140E-02	0.1804E-01	1751400.
20000.	0.4225E-01	0.1067E-02	0.1633E-01	1760000.
25000.	0.4176E-01	0.1009E-02	0.1487E-01	1766800.
30000.	0.4143E-01	0.9590E-03	0.1374E-01	1771300.
VOID = 0.32				
0.	0.3791E-01	0.1427E-02	0.2200E-01	1692900.
100.	0.3787E-01	0.1427E-02	0.2187E-01	1682900.
500.	0.3838E-01	0.1419E-02	0.2189E-01	1677400.
2000.	0.3965E-01	0.1386E-02	0.2196E-01	1673000.
4000.	0.4087E-01	0.1342E-02	0.2174E-01	1670800.
6000.	0.4172E-01	0.1300E-02	0.2134E-01	1670900.
8000.	0.4228E-01	0.1261E-02	0.2084E-01	1672300.
10000.	0.4263E-01	0.1224E-02	0.2027E-01	1674600.
15000.	0.4294E-01	0.1145E-02	0.1882E-01	1681200.
20000.	0.4294E-01	0.1078E-02	0.1741E-01	1688300.
25000.	0.4265E-01	0.1025E-02	0.1616E-01	1694200.
30000.	0.4251E-01	0.9833E-03	0.1515E-01	1698200.
VOID = 0.64				
0.	0.3565E-01	0.1391E-02	0.2116E-01	1611000.
100.	0.3560E-01	0.1391E-02	0.2103E-01	1598800.
500.	0.3620E-01	0.1385E-02	0.2110E-01	1591200.
2000.	0.3779E-01	0.1356E-02	0.2137E-01	1583200.
4000.	0.3942E-01	0.1318E-02	0.2139E-01	1577400.
6000.	0.4064E-01	0.1282E-02	0.2122E-01	1574500.
8000.	0.4155E-01	0.1248E-02	0.2095E-01	1573400.
10000.	0.4222E-01	0.1217E-02	0.2069E-01	1573200.
15000.	0.4319E-01	0.1149E-02	0.1963E-01	1575500.
20000.	0.4367E-01	0.1093E-02	0.1862E-01	1578900.
25000.	0.4393E-01	0.1046E-02	0.1771E-01	1582000.
30000.	0.4413E-01	0.1010E-02	0.1694E-01	1584000.

CROSS SECTION INTERPOLATING TABLE  
TABLE SET 2 BXB HWR UNIT CELL 2.33 W/D

F I S I						T H E R M A I				
EXPOSURE (MWD/MT)	0	ABSORPTION	NU- FISSION	KAPPA- FISSION	DOWN SCATTERING	0	ABSORPTION	NU- FISSION	KAPPA- FISSION	
VOID = 0.0										
0.	0.1362E+01	0.6780E-02	0.4344E-02	0.3442E+00	0.2040E-01	0.3940E+00	0.4538E-01	0.6783E-01	0.5642E+01	
100.	0.1421E+01	0.6802E-02	0.4306E-02	0.3420E+00	0.2044E-01	0.3947E+00	0.4645E-01	0.6641E-01	0.5616E+01	
500.	0.1441E+01	0.6808E-02	0.4269E-02	0.3389E+00	0.2074E-01	0.3949E+00	0.4710E-01	0.6658E-01	0.5603E+01	
2000.	0.1443E+01	0.6810E-02	0.4185E-02	0.3312E+00	0.2080E-01	0.3944E+00	0.4783E-01	0.6672E-01	0.5569E+01	
4000.	0.1445E+01	0.6814E-02	0.4069E-02	0.3210E+00	0.2091E-01	0.3935E+00	0.4848E-01	0.6620E-01	0.5479E+01	
6000.	0.1449E+01	0.6820E-02	0.3955E-02	0.3111E+00	0.2103E-01	0.3925E+00	0.4885E-01	0.6517E-01	0.5356E+01	
8000.	0.1452E+01	0.6829E-02	0.3843E-02	0.3015E+00	0.2115E-01	0.3916E+00	0.4898E-01	0.6380E-01	0.5212E+01	
10000.	0.1455E+01	0.6839E-02	0.3735E-02	0.2923E+00	0.2127E-01	0.3907E+00	0.4894E-01	0.6219E-01	0.5053E+01	
15000.	0.1464E+01	0.6876E-02	0.3493E-02	0.2720E+00	0.2155E-01	0.3888E+00	0.4837E-01	0.5784E-01	0.4647E+01	
20000.	0.1462E+01	0.6924E-02	0.3275E-02	0.2540E+00	0.2181E-01	0.3872E+00	0.4736E-01	0.5317E-01	0.4233E+01	
25000.	0.1450E+01	0.6988E-02	0.3085E-02	0.2387E+00	0.2207E-01	0.3858E+00	0.4625E-01	0.4864E-01	0.3843E+01	
30000.	0.1439E+01	0.7070E-02	0.2927E-02	0.2261E+00	0.2231E-01	0.3848E+00	0.4521E-01	0.4468E-01	0.3502E+01	
VOID = 0.32										
0.	0.1578E+01	0.6571E-02	0.4228E-02	0.3362E+00	0.1630E-01	0.4682E+00	0.4332E-01	0.6528E-01	0.5494E+01	
100.	0.1599E+01	0.6590E-02	0.4213E-02	0.3352E+00	0.1643E-01	0.4617E+00	0.4437E-01	0.6486E-01	0.5467E+01	
500.	0.1600E+01	0.6600E-02	0.4196E-02	0.3336E+00	0.1648E-01	0.4616E+00	0.4508E-01	0.6494E-01	0.5462E+01	
2000.	0.1602E+01	0.6607E-02	0.4121E-02	0.3266E+00	0.1653E-01	0.4610E+00	0.4607E-01	0.6552E-01	0.5461E+01	
4000.	0.1605E+01	0.6618E-02	0.4018E-02	0.3173E+00	0.1662E-01	0.4599E+00	0.4705E-01	0.6553E-01	0.5412E+01	
6000.	0.1608E+01	0.6632E-02	0.3916E-02	0.3083E+00	0.1671E-01	0.4588E+00	0.4772E-01	0.6500E-01	0.5328E+01	
8000.	0.1611E+01	0.6646E-02	0.3817E-02	0.2996E+00	0.1680E-01	0.4577E+00	0.4815E-01	0.6412E-01	0.5221E+01	
10000.	0.1614E+01	0.6663E-02	0.3720E-02	0.2913E+00	0.1689E-01	0.4567E+00	0.4838E-01	0.6298E-01	0.5100E+01	
15000.	0.1622E+01	0.6711E-02	0.3504E-02	0.2729E+00	0.1709E-01	0.4546E+00	0.4838E-01	0.5967E-01	0.4776E+01	
20000.	0.1622E+01	0.6768E-02	0.3308E-02	0.2566E+00	0.1729E-01	0.4528E+00	0.4788E-01	0.5593E-01	0.4435E+01	
25000.	0.1612E+01	0.6836E-02	0.3136E-02	0.2425E+00	0.1746E-01	0.4513E+00	0.4718E-01	0.5219E-01	0.4107E+01	
30000.	0.1602E+01	0.6918E-02	0.2989E-02	0.2308E+00	0.1763E-01	0.4501E+00	0.4647E-01	0.4874E-01	0.3812E+01	
VOID = 0.64										
0.	0.1794E+01	0.6241E-02	0.4106E-02	0.3274E+00	0.1220E-01	0.5523E+00	0.4067E-01	0.6247E-01	0.5257E+01	
100.	0.1796E+01	0.6262E-02	0.4108E-02	0.3277E+00	0.1228E-01	0.5536E+00	0.4168E-01	0.6204E-01	0.5228E+01	
500.	0.1797E+01	0.6274E-02	0.4095E-02	0.3263E+00	0.1232E-01	0.5542E+00	0.4246E-01	0.6226E-01	0.5233E+01	
2000.	0.1799E+01	0.6288E-02	0.4032E-02	0.3202E+00	0.1236E-01	0.5537E+00	0.4378E-01	0.6339E-01	0.5274E+01	
4000.	0.1802E+01	0.6309E-02	0.3945E-02	0.3121E+00	0.1243E-01	0.5525E+00	0.4517E-01	0.6404E-01	0.5274E+01	
6000.	0.1805E+01	0.6330E-02	0.3858E-02	0.3043E+00	0.1249E-01	0.5512E+00	0.4623E-01	0.6413E-01	0.5239E+01	
8000.	0.1808E+01	0.6353E-02	0.3775E-02	0.2968E+00	0.1255E-01	0.5501E+00	0.4702E-01	0.6386E-01	0.5181E+01	
10000.	0.1811E+01	0.6377E-02	0.3695E-02	0.2897E+00	0.1261E-01	0.5490E+00	0.4759E-01	0.6332E-01	0.5107E+01	
15000.	0.1818E+01	0.6441E-02	0.3513E-02	0.2738E+00	0.1274E-01	0.5467E+00	0.4833E-01	0.6135E-01	0.4888E+01	
20000.	0.1817E+01	0.6510E-02	0.3348E-02	0.2598E+00	0.1286E-01	0.5449E+00	0.4850E-01	0.5887E-01	0.4647E+01	
25000.	0.1808E+01	0.6585E-02	0.3202E-02	0.2476E+00	0.1297E-01	0.5433E+00	0.4839E-01	0.5623E-01	0.4406E+01	
30000.	0.1800E+01	0.6669E-02	0.3075E-02	0.2374E+00	0.1306E-01	0.5421E+00	0.4816E-01	0.5369E-01	0.4182E+01	

TIME STEP # 1 << EDWIN HATCH UNIT #2 100% POWER 7-11-77 >> PAGE 8

CROSS SECTION INTERPOLATING TABLE

TABLE SET 2 BXH BWR UNIT CELL 2.33 W/O

EXPOSURE (MWD/MT)	ABSORPTION XF REMOVED	FAST FISSION	THERMAL FISSION	XF-135 MICRO ABSORPTION
*****	*****	*****	*****	*****
VOID = 0.0				
0.	0.4538E-01	0.1678E-02	0.2758E-01	1704000.
100.	0.4525E-01	0.1667E-02	0.2739E-01	1695000.
500.	0.4562E-01	0.1651E-02	0.2728E-01	1690800.
2000.	0.4635E-01	0.1611E-02	0.2703E-01	1690200.
4000.	0.4700E-01	0.1557E-02	0.2648E-01	1691700.
6000.	0.4739E-01	0.1505E-02	0.2578E-01	1696800.
8000.	0.4755E-01	0.1454E-02	0.2499E-01	1698700.
10000.	0.4753E-01	0.1405E-02	0.2415E-01	1703600.
15000.	0.4702E-01	0.1297E-02	0.2200E-01	1715300.
20000.	0.4610E-01	0.1202E-02	0.1986E-01	1727900.
25000.	0.4508E-01	0.1120E-02	0.1787E-01	1739600.
30000.	0.4414E-01	0.1052E-02	0.1613E-01	1749400.
VOID = 0.32				
0.	0.4332E-01	0.1639E-02	0.2686E-01	1640600.
100.	0.4318E-01	0.1633E-02	0.2667E-01	1630000.
500.	0.4362E-01	0.1625E-02	0.2660E-01	1624600.
2000.	0.4460E-01	0.1588E-02	0.2650E-01	1622200.
4000.	0.4558E-01	0.1539E-02	0.2614E-01	1622000.
6000.	0.4626E-01	0.1491E-02	0.2563E-01	1623700.
8000.	0.4671E-01	0.1445E-02	0.2503E-01	1626500.
10000.	0.4695E-01	0.1401E-02	0.2436E-01	1630200.
15000.	0.4700E-01	0.1302E-02	0.2261E-01	1640300.
20000.	0.4657E-01	0.1215E-02	0.2082E-01	1651500.
25000.	0.4595E-01	0.1139E-02	0.1912E-01	1662200.
30000.	0.4531E-01	0.1076E-02	0.1761E-01	1671600.
VOID = 0.64				
0.	0.4067E-01	0.1595E-02	0.2571E-01	1550400.
100.	0.4052E-01	0.1596E-02	0.2550E-01	1537500.
500.	0.4103E-01	0.1589E-02	0.2548E-01	1530300.
2000.	0.4234E-01	0.1556E-02	0.2558E-01	1525000.
4000.	0.4372E-01	0.1513E-02	0.2546E-01	1521800.
6000.	0.4478E-01	0.1471E-02	0.2518E-01	1520900.
8000.	0.4557E-01	0.1431E-02	0.2480E-01	1521500.
10000.	0.4615E-01	0.1392E-02	0.2436E-01	1523100.
15000.	0.4692E-01	0.1306E-02	0.2312E-01	1529200.
20000.	0.4714E-01	0.1231E-02	0.2181E-01	1536900.
25000.	0.4708E-01	0.1165E-02	0.2053E-01	1544800.
30000.	0.4690E-01	0.1108E-02	0.1937E-01	1551900.

TIME STEP # 1 << EDWIN HATCH UNIT #2 100% POWER 7-11-77 >> PAGE 9

\*\*\*\*\* Xe-135 + I-135 FISSION YIELDS \*\*\*\*\*

EXPOSURE  
(MWD/MT) TABLE SET = 1 2 3

0.	0.0660	0.0660	0.0661
100.	0.0661	0.0661	0.0664
500.	0.0666	0.0665	0.0683
2000.	0.0674	0.0670	0.0696
4000.	0.0681	0.0679	0.0706
6000.	0.0686	0.0680	0.0713
8000.	0.0691	0.0686	0.0717
10000.	0.0698	0.0691	0.0723
15000.	0.0706	0.0698	0.0727
20000.	0.0712	0.0705	0.0730
25000.	0.0718	0.0711	0.0731
30000.	0.0722	0.0716	0.0732



TIME STEP # 1 << EDWIN HATCH UNIT 02 100% POWER 7-11-77 >> PAGE 10

\*\*\* FUEL TYPE EDIT \*\*\*

FTYPE (1,J)

	1	2	3	4	5	6	7	8	9	10	11	12	13
1													
2	1	1	1	1	1	1	1	1	1	2	2	2	3
3	2	1	1	2	1	2	1	2	1	2	2	2	3
4	3	1	2	1	2	1	2	1	2	1	2	2	3
5	4	1	1	2	1	2	1	2	1	2	2	2	3
6	5	1	2	1	2	1	2	1	2	2	2	2	3
7	6	1	1	2	1	2	1	2	1	2	2	2	3
8	7	1	2	1	2	1	2	1	2	2	2	2	3
9	8	1	1	2	1	2	1	2	2	2	2	2	3
10	9	1	2	1	2	2	2	2	2	2	2	2	3
11	10	2	2	2	2	2	2	2	2	2	2	2	3
12	11	2	2	2	2	2	2	3	3	3			
13	12	2	2	2	2	2	3						
14	13	3	3	3	3	1							

TIME STEP # 1      << EDWIN HATCH UNIT #2    100% POWER      7-11-77      22      PAGE 11

\*\*\* CONTROL ROD POSITIONS \*\*\*

NOTCH (1, J)

	1	2	3	4	5	6	7	8	9	10	11	12	13
1	23	0	0	23	23	0	0	23	23	0	0	12	12
2	0	16	16	0	0	16	16	0	0	16	16	0	0
3	0	16	16	0	0	16	16	0	0	16	16	0	0
4	23	0	0	23	23	0	0	23	23	0	0	0	0
5	23	0	0	23	23	0	0	23	23	0	0	0	0
6	0	16	16	0	0	16	16	0	0	16	16	0	0
7	0	16	16	0	0	16	16	0	0	16	16	0	0
8	23	0	0	23	23	0	0	23	23	0	0	0	0
9	23	0	0	23	23	0	0	23	23	0	0	0	0
10	0	16	16	0	0	16	16	0	0	0	0	0	0
11	0	16	16	0	0	16	16	0	0	0	0	0	0
12	12	0	0	0	0	0	0	0	0	0	0	0	0
13	12	0	0	0	0	0	0	0	0	0	0	0	0

TIME STEP # 1

&lt;&lt; EDWIN HATCH UNIT #2 100% POWER

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&gt;&gt;

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\*\*\* CONTROL ROD WORTH FOR TABLE SET 1 \*\*\*

EXPOSURE (MW/MT)	WORTH
---------------------	-------

VOID = 0.0

0.0	0.24
0.10E+03	0.24
0.50E+03	0.24
0.20E+04	0.24
0.40E+04	0.24
0.60E+04	0.24
0.80E+04	0.24
0.10E+05	0.24
0.15E+05	0.24
0.20E+05	0.24
0.25E+05	0.24
0.30E+05	0.24

VOID = 0.32

0.0	0.27
0.10E+03	0.27
0.50E+03	0.27
0.20E+04	0.27
0.40E+04	0.27
0.60E+04	0.27
0.80E+04	0.27
0.10E+05	0.27
0.15E+05	0.27
0.20E+05	0.27
0.25E+05	0.27
0.30E+05	0.27

VOID = 0.64

0.0	0.32
0.10E+03	0.32
0.50E+03	0.32
0.20E+04	0.32
0.40E+04	0.32
0.60E+04	0.32
0.80E+04	0.32
0.10E+05	0.32
0.15E+05	0.32
0.20E+05	0.32
0.25E+05	0.32
0.30E+05	0.32

TIME STEP # 1

&lt;&lt; EDWIN HATCH UNIT #2 100% POWER

7-11-77

&gt;&gt;

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\*\*\* CONTROL ROD WORTH FOR TABLE SET 2 \*\*\*

EXPOSURE (MIN/MT)	WORTH
----------------------	-------

VOID = 0.0

0.0	0.24
0.10E+03	0.24
0.50E+03	0.24
0.20E+04	0.24
0.40E+04	0.24
0.60E+04	0.24
0.80E+04	0.24
0.10E+05	0.24
0.15E+05	0.24
0.20E+05	0.24
0.25E+05	0.24
0.30E+05	0.24

VOID = 0.32

0.0	0.27
0.10E+03	0.27
0.50E+03	0.27
0.20E+04	0.27
0.40E+04	0.27
0.60E+04	0.27
0.80E+04	0.27
0.10E+05	0.27
0.15E+05	0.27
0.20E+05	0.27
0.25E+05	0.27
0.30E+05	0.27

VOID = 0.64

0.0	0.32
0.10E+03	0.32
0.50E+03	0.32
0.20E+04	0.32
0.40E+04	0.32
0.60E+04	0.32
0.80E+04	0.32
0.10E+05	0.32
0.15E+05	0.32
0.20E+05	0.32
0.25E+05	0.32
0.30E+05	0.32

TIME STEP # 1 << EDWIN HATCH UNIT #2 100% POWER 7-11-77 >> PAGE 14

\*\*\* CONTROL ROD WORTH FOR TABLE SET 3 \*\*\*

EXPOSURE (MIN/MT)	WORTH
*****	*****
VOID = 0.0	
0.0	0.24
0.10E+03	0.24
0.50E+03	0.24
0.20E+04	0.24
0.40E+04	0.24
0.60E+04	0.24
0.80E+04	0.24
0.10E+05	0.24
0.15E+05	0.24
0.20E+05	0.24
0.25E+05	0.24
0.30E+05	0.24
VOID = 0.32	
0.0	0.27
0.10E+03	0.27
0.50E+03	0.27
0.20E+04	0.27
0.40E+04	0.27
0.60E+04	0.27
0.80E+04	0.27
0.10E+05	0.27
0.15E+05	0.27
0.20E+05	0.27
0.25E+05	0.27
0.30E+05	0.27
VOID = 0.64	
0.0	0.32
0.10E+03	0.32
0.50E+03	0.32
0.20E+04	0.32
0.40E+04	0.32
0.60E+04	0.32
0.80E+04	0.32
0.10E+05	0.32
0.15E+05	0.32
0.20E+05	0.32
0.25E+05	0.32
0.30E+05	0.32

TIME STEP # 1 EDWIN MATCH UNIT #2 100% POWER

7-11-77

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\* THERMAL-HYDRAULIC DATA FOR THIS TIME STEP \*

CORE POWER	2436.0	MW-THERMAL
AV. HEAT FLUX	145060.	BTU/HR/FT**2
PEAKING FACTOR	1.05	
SAT. VAP. SPEC. VOL.	0.4297	FT**3/LBM
SAT. LIQ. SPEC. VOL.	0.02170	FT**3/LBM
INLET ENTHALPY	526.90	BTU/LBM
HEAT OF VAPORIZATION	642.80	BTU/LBM
ENTHALPY OF SAT. LIQ.	547.80	BTU/LBM
CORE PRESSURE	1035.0	PSIA
INCORE FLOW	7.700E+07	LBM/HR
FLOW AREA PER CHANNEL	0.1099	FT**2

ARMAND CORRELATION COEFFICIENTS

AC1 = 0.433  
AC2 = 0.167

COOLANT FLOW VARIATION WITH POWER

P = RELATIVE POWER IN CHANNEL  
FLOW = RELATIVE COOLANT FLOW IN CHANNEL (UNNORMALIZED)

CHANNEL TYPE  
\*\*\*\*\*

1	FLOW = 0.100E+01 + 0.0	*P + 0.0	*P**2
2	FLOW = 0.500E+00 + 0.0	*P + 0.0	*P**2

TIME STEP # 1 << EDWIN HATCH UNIT #2 100% POWER 7-11-77 >> PAGE 16

\*\*\* CHANNEL TYPE EDIT \*\*\*

CTYPE (I,J)

J= 1 2 3 4 5 6 7 8 9 10 11 12 13

1

1 1 1 1 1 1 1 1 1 1 1 1 2

2 1 1 1 1 1 1 1 1 1 1 1 2

3 1 1 1 1 1 1 1 1 1 1 1 2

4 1 1 1 1 1 1 1 1 1 1 1 2

5 1 1 1 1 1 1 1 1 1 1 1 2

6 1 1 1 1 1 1 1 1 1 1 1 2

7 1 1 1 1 1 1 1 1 1 1 2

8 1 1 1 1 1 1 1 1 1 1 2

9 1 1 1 1 1 1 1 1 1 1 2

10 1 1 1 1 1 1 1 1 1 2

11 1 1 1 1 1 1 1 1 2

12 1 1 1 1 1 2

13 2 2 2 2 2

\*\*\*\*\*  
ITERATION MONITOR  
\*\*\*\*\*

VOID ITERATION	OUTER ITERATION	INNER ITERATION	K-EFF	TEST0	TESTP
-------------------	--------------------	--------------------	-------	-------	-------

1	1	1	1.0322	0.312E-01	
	2	3	1.0530	0.197E-01	
	3	3	1.0704	0.163E-01	
	4	3	1.0798	0.876E-02	
	5	3	1.0837	0.357E-02	
	6	3	1.0852	0.136E-02	

2	1	3	1.0455	0.379E-01	
	2	3	1.0463	0.818E-03	
	3	3	1.0442	0.202E-02	
	4	3	1.0432	0.997E-03	
	5	3	1.0432	0.143E-04	
	6	3	1.0436	0.441E-03	0.561E+00

3	1	3	1.0450	0.132E-02	
	2	3	1.0454	0.351E-03	
	3	3	1.0459	0.521E-03	
	4	3	1.0463	0.392E-03	
	5	3	1.0466	0.285E-03	
	6	3	1.0469	0.214E-03	0.309E+00

4	1	3	1.0498	0.278E-02	
	2	3	1.0493	0.443E-03	
	3	3	1.0492	0.887E-04	
	4	3	1.0492	0.304E-04	
	5	3	1.0493	0.874E-04	
	6	3	1.0495	0.117E-03	0.144E+00

5	1	3	1.0523	0.268E-02	
	2	3	1.0519	0.325E-03	
	3	3	1.0519	0.114E-04	
	4	3	1.0520	0.544E-04	
	5	3	1.0521	0.709E-04	
	6	3	1.0521	0.861E-04	0.127E+00

6	1	3	1.0546	0.232E-02	
	2	3	1.0543	0.273E-03	
	3	3	1.0543	0.191E-04	
	4	3	1.0543	0.333E-04	
	5	3	1.0543	0.276E-04	
	6	3	1.0544	0.641E-04	0.135E+00

7	1	3	1.0563	0.182E-02	
	2	3	1.0561	0.196E-03	
	3	3	1.0561	0.124E-04	
	4	3	1.0561	0.272E-04	
	5	3	1.0562	0.274E-04	
	6	3	1.0562	0.162E-04	0.140E+00



	1	3	1.0575	0.134E-03	
	2	3	1.0575	0.477E-05	
	3	3	1.0575	0.241E-04	
	4	3	1.0575	0.721E-05	0.105E+00
9	5	3	1.0583	0.756E-03	
	1	3	1.0582	0.515E-04	
	2	3	1.0582	0.114E-04	
	3	3	1.0582	0.596E-05	
	4	3	1.0582	0.668E-05	0.936E-01
10	5	3	1.0588	0.525E-03	
	1	3	1.0587	0.296E-04	
	2	3	1.0588	0.328E-05	
	3	3	1.0588	0.596E-05	
	4	3	1.0588	0.954E-06	0.824E-01
11	5	3	1.0591	0.371E-03	
	1	3	1.0591	0.143E-04	
	2	3	1.0591	0.191E-05	
	3	3	1.0591	0.381E-05	
	4	3	1.0591	0.572E-05	0.701E-01
12	5	3	1.0594	0.255E-03	
	1	3	1.0594	0.763E-05	
	2	3	1.0594	0.757E-05	
	3	3	1.0594	0.381E-05	
	4	3	1.0594	0.668E-05	0.579E-01
13	5	3	1.0596	0.171E-03	
	1	3	1.0596	0.685E-05	
	2	3	1.0596	0.381E-05	
	3	3	1.0596	0.417E-05	
	4	3	1.0596	0.477E-05	0.471E-01
14	5	3	1.0597	0.105E-03	
	1	3	1.0597	0.471E-05	
	2	3	1.0597	0.0	0.213E-01
15	3	3	1.0597	0.346E-04	
	1	3	1.0597	0.151E-04	
	2	3	1.0597	0.668E-05	
	3	3	1.0597	0.0	0.240E-01
16	4	3	1.0597	0.280E-04	
	1	3	1.0597	0.650E-05	
	2	3	1.0598	0.721E-05	
	3	3	1.0597	0.572E-05	
	4	3	1.0597	0.417E-06	0.271E-01
17	5	3	1.0598	0.208E-04	
	1	3	1.0598	0.829E-05	
	2	3	1.0598	0.417E-06	0.121E-01
18	3	3	1.0598	0.526E-05	
	1	3	1.0598	0.136E-04	
	2	3	1.0598	0.0	0.937E-02
19	3	3	1.0598	0.668E-05	
	1	3	1.0598	0.111E-04	
	2	3	1.0598	0.0	0.803E-02
20	3	3	1.0598	0.954E-06	0.311E-02
21	1	3	1.0598	0.286E-05	

	2	3	1.0598	0.417E-06	0.286E-02
22	1	3	1.0598	0.954E-06	0.217E-02
23	1	3	1.0598	0.507E-05	
	2	3	1.0598	0.572E-05	
	3	3	1.0598	0.435E-05	
	4	3	1.0598	0.167E-05	0.671E-02
24	1	3	1.0598	0.954E-05	
	2	3	1.0598	0.918E-05	
	3	3	1.0598	0.310E-05	
	4	3	1.0598	0.0	0.668E-02
25	1	3	1.0598	0.954E-05	
	2	3	1.0598	0.381E-05	
	3	3	1.0598	0.703E-05	
	4	3	1.0598	0.381E-05	0.504E-02
26	1	3	1.0598	0.668E-05	
	2	3	1.0598	0.954E-06	0.193E-02
27	1	3	1.0598	0.131E-05	
	2	3	1.0598	0.191E-05	
	3	3	1.0598	0.507E-05	
	4	3	1.0598	0.858E-05	0.293E-02
28	1	3	1.0598	0.286E-05	
	2	3	1.0598	0.775E-06	0.123E-02
29	1	3	1.0598	0.0	0.525E-03

TIME STEP # 1 << EDWIN MATCH UNIT #2 100% POWER 7=11=77 >> PAGE 18  
 EXPOSURE = 0. MWD/MT

\*\*\* INTEGRATED POWER EDIT \*\*\*

P (I,J)

J=	1	2	3	4	5	6	7	8	9	10	11	12	13
1	0.736	1.166	1.194	0.782	0.794	1.242	1.259	0.829	0.865	1.496	1.472	1.082	0.304
2	1.166	1.099	1.212	1.261	1.372	1.161	1.260	1.320	1.452	1.307	1.232	1.122	0.321
3	1.194	1.212	1.157	1.387	1.298	1.269	1.187	1.428	1.338	1.273	1.198	1.098	0.313
4	0.782	1.261	1.387	0.832	0.899	1.312	1.413	0.839	0.899	1.390	1.342	1.055	0.288
5	0.794	1.372	1.298	0.899	0.837	1.402	1.290	0.872	0.850	1.283	1.188	0.871	0.210
6	1.242	1.161	1.269	1.312	1.402	1.151	1.205	1.211	1.237	0.972	0.755	0.268	
7	1.259	1.260	1.188	1.413	1.290	1.206	1.082	1.245	1.129	0.791	0.262		
8	0.829	1.320	1.428	0.840	0.872	1.211	1.245	0.732	0.610	0.688	0.197		
9	0.865	1.452	1.338	0.900	0.850	1.237	1.130	0.610	0.463	0.503	0.121		
10	1.496	1.307	1.273	1.390	1.284	0.973	0.743	0.688	0.583	0.153			
11	1.472	1.232	1.198	1.342	1.189	0.757	0.276	0.201	0.120				
12	1.082	1.122	1.098	1.055	0.871	0.268							
13	0.304	0.321	0.313	0.288	0.210								

TIME STEP # 1 EDWIN HATCH UNIT #2 100% POWER  
EXPOSURE # 0. MWD/MT

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AXIAL AVERAGES  
=====

K POWER VOID

1	0.215	0.0
2	0.303	0.0
3	0.520	0.0
4	0.626	0.0
5	0.699	0.0
6	0.738	0.00
7	0.735	0.02
8	0.708	0.05
9	0.678	0.08
10	0.658	0.11
11	0.657	0.15
12	0.690	0.18
13	0.770	0.22
14	0.889	0.26
15	1.072	0.30
16	1.346	0.35
17	1.887	0.40
18	2.081	0.45
19	2.074	0.50
20	1.926	0.53
21	1.677	0.56
22	1.360	0.58
23	0.994	0.59
24	0.621	0.60

PEAK POWER = 3.307 AT LOCATION (10, 2, 18)

PEAK ASSEMBLY INTEGRATED POWER = 1.496 AT LOCATION (10, 1)

TIME STEP # 1 SS EDWIN HATCH UNIT #2 100% POWER  
 EXPOSURE = 0. MWD/MT

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\*\*\* AVERAGE EXPOSURE EDIT \*\*\*

EX (I,J)

	J=	1	2	3	4	5	6	7	8	9	10	11	12	13
1														
1	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
2	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
3	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
4	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
5	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
6	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
7	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
8	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
9	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
10	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
11	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
12	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
13	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0

TIME STEP # 1 << EDWIN HATCH UNIT #2 100% POWER

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\*\*\* RELATIVE FLOW EDIT \*\*\* TOTAL FLOW = 9.170E+08 LBM/HR

ELOW (I,J)

	1	2	3	4	5	6	7	8	9	10	11	12	13
1	1.06	1.06	1.06	1.06	1.06	1.06	1.06	1.06	1.06	1.06	1.06	1.06	0.53
2	1.06	1.06	1.06	1.06	1.06	1.06	1.06	1.06	1.06	1.06	1.06	1.06	0.53
3	1.06	1.06	1.06	1.06	1.06	1.06	1.06	1.06	1.06	1.06	1.06	1.06	0.53
4	1.06	1.06	1.06	1.06	1.06	1.06	1.06	1.06	1.06	1.06	1.06	1.06	0.53
5	1.06	1.06	1.06	1.06	1.06	1.06	1.06	1.06	1.06	1.06	1.06	1.06	0.53
6	1.06	1.06	1.06	1.06	1.06	1.06	1.06	1.06	1.06	1.06	1.06	0.53	
7	1.06	1.06	1.06	1.06	1.06	1.06	1.06	1.06	1.06	1.06	0.53		
8	1.06	1.06	1.06	1.06	1.06	1.06	1.06	1.06	1.06	1.06	0.53		
9	1.06	1.06	1.06	1.06	1.06	1.06	1.06	1.06	1.06	1.06	0.53		
10	1.06	1.06	1.06	1.06	1.06	1.06	1.06	1.06	1.06	0.53			
11	1.06	1.06	1.06	1.06	1.06	1.06	1.06	1.06	0.53				
12	1.06	1.06	1.06	1.06	1.06	0.53							
13	0.53	0.53	0.53	0.53	0.53								

TIME STEP # 1 << EDWIN MATCH UNIT #2 100% POWER 7-11-77 >> PAGE 22

\*\*\* VOID ERACTION EDIT \*\*\*

VOID (11.J.241)

	1	2	3	4	5	6	7	8	9	10	11	12	13
1	0.54	0.67	0.67	0.56	0.56	0.68	0.68	0.58	0.59	0.72	0.72	0.65	0.48
2	0.67	0.65	0.67	0.68	0.70	0.66	0.68	0.69	0.72	0.69	0.68	0.66	0.58
3	0.67	0.67	0.66	0.71	0.69	0.69	0.67	0.71	0.70	0.69	0.67	0.65	0.49
4	0.56	0.68	0.71	0.58	0.60	0.69	0.71	0.58	0.60	0.71	0.70	0.64	0.46
5	0.56	0.70	0.69	0.60	0.58	0.71	0.69	0.59	0.58	0.69	0.67	0.59	0.34
6	0.68	0.66	0.69	0.69	0.71	0.66	0.67	0.68	0.68	0.62	0.55	0.43	
7	0.68	0.68	0.67	0.71	0.69	0.67	0.65	0.68	0.66	0.56	0.42		
8	0.58	0.69	0.71	0.58	0.59	0.68	0.68	0.54	0.48	0.52	0.31		
9	0.59	0.72	0.70	0.60	0.58	0.68	0.66	0.48	0.38	0.41	0.08		
10	0.72	0.69	0.69	0.71	0.69	0.62	0.56	0.52	0.41	0.20			
11	0.72	0.68	0.67	0.70	0.67	0.55	0.14	0.0	0.07				
12	0.65	0.66	0.65	0.64	0.59	0.43							
13	0.48	0.58	0.49	0.46	0.34								

TIME STEP # 1 << EDWIN HAICH UNIT #2 100% POWER 7-11-77 >> PAGE 23

\*\*\* MINIMUM CRITICAL HEAT FLUX RATIO EDIT \*\*\*

MCHFR (1.0)

	1	2	3	4	5	6	7	8	9	10	11	12	13
1													
1	4.32	2.12	2.04	4.22	4.22	1.93	1.89	4.08	3.90	1.30	1.34	2.39	7.45
2	2.12	2.03	1.73	1.88	1.58	1.89	1.64	1.75	1.41	1.54	1.72	2.37	7.90
3	2.04	1.73	1.89	1.55	1.79	1.62	1.84	1.47	1.73	1.64	1.85	2.56	8.54
4	4.22	1.88	1.55	4.07	3.69	1.78	1.52	4.17	3.86	1.64	1.78	2.96	9.96
5	4.22	1.58	1.79	3.69	4.13	1.55	1.87	4.03	4.30	2.01	2.37	4.88	13.74
6	1.93	1.89	1.62	1.78	1.55	1.97	1.83	2.18	2.16	2.72	3.51	10.00	
7	1.89	1.64	1.84	1.52	1.87	1.83	2.22	2.09	2.60	3.39	7.86		
8	4.08	1.75	1.47	4.17	4.03	2.18	2.09	5.16	6.16	5.04	12.89		
9	3.90	1.41	1.73	3.86	4.30	2.16	2.60	6.16	8.01	7.19	22.59		
10	1.29	1.54	1.64	1.64	2.01	2.71	3.38	5.04	7.20	18.61			
11	1.34	1.72	1.85	1.78	2.37	3.51	8.72	14.94	22.94				
12	2.39	2.37	2.56	2.96	4.07	9.99							
13	7.45	7.89	8.54	9.95	13.73								

MINIMUM CRITICAL HEAT FLUX RATIO = 1.295 AT LOCATION ( 10, 1, 21 )

\*\*\* END OF CASE \*\*\*



## APPENDIX B

Source Listing

C=====	A	10
C=====	A	20
C	A	30
PROGRAM STORM (INPUT,OUTPUT,TAPE5=INPUT,TAPE6=OUTPUT,TAPE7=OUTPUT)	A	40
C	A	50
C A THREE DIMENSIONAL BWR SIMULATION PROGRAM USING SIMULATED TWO-GROUP	A	60
C DIFFUSION THEORY WITH THERMAL-HYDRAULIC FEEDBACK.	A	70
C	A	80
C-----	A	90
C	A	100
C VERSION 77222	A	110
C	A	120
C DIMENSIONING INSTRUCTIONS	A	130
C =====	A	140
C	A	150
C REPLACE ALL OCCURENCES OF N01 THROUGH N11 WITH THE VALUE INDICATED	A	160
C BELOW. GEOMETRIC DIMENSIONS MUST BE EXACTLY THOSE OF THE PROBLEM	A	170
C TO BE SOLVED. TABLE SET DIMENSIONS (N07-N11) MAY BE EQUAL TO OR	A	180
C GREATER THAN THE ACTUAL DIMENSIONS OF THE TABLES USED.	A	190
C	A	200
C N01 = NUMBER OF NODES IN ENTIRE PROBLEM	A	210
C N02 = NUMBER OF NODES PER PLANE	A	220
C N03 = N01-N02	A	230
C N04 = MAXIMUM NUMBER OF NODES PER ROW	A	240
C N05 = N04-1	A	250
C N06 = NUMBER OF AXIAL NODES	A	260
C N07 = NUMBER OF EXPOSURE POINTS USED IN TABLE SETS	A	270
C N08 = NUMBER OF VOID POINTS USED IN TABLE SETS	A	280
C N09 = NUMBER OF TABLE SETS	A	290
C N10 = N07*N08*N09	A	300
C N11 = NUMBER OF FUEL TYPES	A	310
C	A	320
C FOR CONVERSION TO IBM COMPUTERS CHANGE END OF FILE (EOF) CHECKS AND	A	330
C CHANGE PROGRAM CARD TO COMMENT CARD.	A	340
C	A	350
C-----	A	360

C			A	370
C	DATA SET	USE	A	380
C	-----	-----	A	390
C	5	INPUT (CARD READER)	A	400
C	6	OUTPUT (PRINTER)	A	410
C	7	PUNCH	A	420
C			A	430
C	-----	-----	A	440
C			A	450
	COMMON /HEADR /	NTITLE(20), AVEXP, NTSTEP	A	460
	COMMON /CSINP /	BUMASK(N07), VMASK(N08), DFI(N10), DTI(N10)	A	470
	\$	SAFI(N10), SATI(N10), NSFFI(N10), NSFTI(N10),	A	480
	\$	SDSI(N10), IEXPTS, IVDPTS, ITSETS,	A	490
	\$	KSFTI(N10), KSFFI(N10),	A	500
	\$	DIMTS, DIMEX, DIMVD, NOTCH(N02),	A	510
	\$	TABSET(N01), WORTH(N09,N07,N08)	A	520
	INTEGER	DIMTS, DIMEX, DIMVD, TABSET	A	530
	REAL	NSFFI, NSFTI, KSFFI, KSFTI	A	540
	COMMON /FUEL /	FTYPE(N02), BU(N01), VOID(N01)	A	550
	INTEGER	FTYPE	A	560
	COMMON /NODAL /	DF (N01), DT (N01), KSFF (N01)	A	570
	\$,	KSFT (N01), TFR (N01), NSF1G (N01)	A	580
	\$,	C2 (N01), C4 (N01), C6 (N03)	A	590
	\$,	C7 (N01)	A	600
	REAL	KSFF, KSFT, NSF1G	A	610
	COMMON /GEOM /	NODES, NP, NDSMNP, NX	A	620
	\$,	NZ, NXM1, GX, GZ	A	630
	\$,	NCOL (N04)	A	640
	COMMON /BC /	AF1, AF3, AF5, AF6, AFQ(N04),	A	650
	\$	BF1, BF3, BF5, BF6, BFQ(N04),	A	660
	\$	AT1, AT3, AT5, AT6, ATQ(N04),	A	670
	\$	BT1, BT3, BT5, BT6, BTQ(N04)	A	680
	EQUIVALENCE	(AFQ(1), AF2), (AFQ(2), AF4),	A	690
	\$	(BFQ(1), BF2), (BFQ(2), BF4),	A	700
	\$	(ATQ(1), AT2), (ATQ(2), AT4),	A	710
	\$	(BTQ(1), BT2), (BTQ(2), BT4)	A	720

COMMON /CONVRG/	MAXI, MAXO, MAXP, EPSO, EPSP, ALPHA, SOR	A	730
LOGICAL SOR		A	740
COMMON /FLUX /	POWER(N01), FASTA(N01), THERMA(N01), F(N01)	A	750
COMMON /PROB /	INTTYP, QTRKOR, NOHAL, NOCROD, NOVOID, IEDIT(6)	A	760
\$,	NOXE, FIRST	A	770
LOGICAL QTRKOR,	NOHAL, NOCROD, NOVOID, NOXE, FIRST	A	780
COMMON /WEIGHT/	R, CF, CT, RCF, RCT, B1N, B2N	A	790
COMMON /THERMO/	HFG, XIN, AC1, AC2, AC3, AREA, AVFLUX, AVPWR, PSIA	A	800
\$,	B1(4), B2(4), B3(4), CTYPE(N02), FLOW(N02), TFLOW	A	810
\$,	PCTPWR	A	820
INTEGER CTYPE		A	830
COMMON /XENON /	PD, PRSA1(N10), PRSA2(N10), SAXE(N10)	A	840
C		A	850
NAMELIST /INPUT/	IPTYPE, ICNTRL, IEDIT, IPDRD, IEXRD, TONNES,	A	860
\$	PCTPWR, MWTH, IGEOM, DX, DZ, A1,	A	870
\$	A2, NCOL, IEXPTS, IVDPTS, ITSETS, BUMASK,	A	880
\$	VMASK, MAXI, MAXO, MAXP, ALPHA, ISOR,	A	890
\$	EPSO, EPSP, AVHFLX, VG, VF, HIN,	A	900
\$	HSAT, HFG, PSIA, AC1, AC2, IFLOW,	A	910
\$	TFLOW, AREA, PEAKF, NBANKS, IXE, ITSAS,	A	920
\$	IFTYPS	A	930
REAL MWTH		A	940
COMMON /CROD /	NRODS(6,2), RODIND(N02)	A	950
INTEGER RODIND		A	960
DIMENSION ASMEX(N06),	WORD(20), YIX(N09,N07)	A	970
INTEGER TSOVLY(N11,N06)		A	980
DATA A1/ .31/, A2/ .73/,	PEAKF/ 1.05/, MWTH/0./, IFTYPS/1/,	A	990
\$	TONNES/0./, IGEOM/1/, ICNTRL/0/, IPDRD/0/, IEXRD/0/	A	1000
\$,	NBANKS/0/, IFLOW/0/, ISOR/0/, IXE/1/, IPTYPE/1/, ITSAS/2/	A	1010
C		A	1020
C-----		A	1030
C		A	1040
READ (5,1000) NTITLE		A	1050
CALL HEADER (1)		A	1060
C		A	1070
C READ PROBLEM DESCRIPTION		A	1080

READ (5, INPUT)	A	1090
IF ( IPTYPE .EQ. 2 ) NOHAL = .FALSE.	A	1100
IF ( ICNTRL .NE. 0 ) NOCROD = .FALSE.	A	1110
IF ( IGEOM .EQ. 2 ) QTRKOR = .FALSE.	A	1120
IF ( ISOR .EQ. 1 ) SOR = .TRUE.	A	1130
IF ( IXE .EQ. 1 ) NOXE = .FALSE.	A	1140
C	A	1150
CHECK FOR INPUT GEOMETRY ERRORS	A	1160
IF ( NCOL(1) .NE. NX) CALL ERROR (2)	A	1170
ISUM = 0	A	1180
DO 222 I=1,NX	A	1190
222 ISUM = ISUM+NCOL(I)	A	1200
IF ( ISUM .NE. NP) CALL ERROR (3)	A	1210
IF ( NZ*NP .NE. NODES) CALL ERROR (4)	A	1220
C	A	1230
C READ B.C. DATA	A	1240
IF ( QTRKOR ) GO TO 111	A	1250
READ (5,*) AF1,BF1,AF2,BF2,AF3,BF3,AF4,BF4,AF5,BF5,AF6,BF6	A	1260
\$ ,AT1,BT1,AT2,BT2,AT3,BT3,AT4,BT4,AT5,BT5,AT6,BT6	A	1270
WRITE (6,4010) DX,DZ,A1,A2	A	1280
WRITE (6,4020) AF1,BF1,AT1,BT1,AF2,BF2,AT2,BT2,AF3,BF3,AT3,BT3,	A	1290
\$ AF4,BF4,AT4,BT4,AF5,BF5,AT5,BT5,AF6,BF6,AT6,BT6	A	1300
GO TO 112	A	1310
111 AF1 = 1.	A	1320
BF1 = 0.	A	1330
AF3 = 1.	A	1340
BF3 = 0.	A	1350
AT1 = 1.	A	1360
BT1 = 0.	A	1370
AT3 = 1.	A	1380
BT3 = 0.	A	1390
READ (5,*) AF5,BF5,AF6,BF6	A	1400
\$ ,AT5,BT5,AT6,BT6	A	1410
\$ ,(AFQ(I),BFQ(I), I=1,NX)	A	1420
\$ ,(ATQ(I),BTQ(I), I=1,NX)	A	1430
WRITE (6,4010) DX, DZ, A1, A2	A	1440

WRITE (6,4022) AF5,BF5,AT5,BT5,AF6,BF6,AT6,BT6,	A	1450
5 (I, AFQ(I), BFQ(I), ATQ(I), BTQ(I), I=1,Nx)	A	1460
C	A	1470
112 WRITE (6,4000) EPS0,EPSP,MAXI,MAX0,MAXP	A	1480
C	A	1490
CHECK INTERPOLATING TABLE DESCRIPTION	A	1500
INTTYP = 1	A	1510
IF ( IVDPTS .GT. 1) INTTYP = 3	A	1520
IF ( IEXPTS .GT. 1) INTTYP = INTTYP+1	A	1530
IF ( INTTYP .GT. 2) NOVOID = .FALSE.	A	1540
IF ( IPTYPE .EQ. 2 .AND. IEXPTS .EQ. 1 ) CALL ERROR (5)	A	1550
IF ( ITSETS .GT. DIMTS) CALL ERROR (6)	A	1560
IF ( IEXPTS .GT. DIMEX) CALL ERROR (7)	A	1570
IF ( IVDPTS .GT. DIMVD) CALL ERROR (8)	A	1580
C	A	1590
CHECK FOR ERRORS IN MASKS	A	1600
IF ( IEXPTS .LE. 1) GO TO 114	A	1610
DO 113 I=2,IEXPTS	A	1620
IF ( BUMASK(I-1) .GT. BUMASK(I) ) CALL ERROR (9)	A	1630
113 CONTINUE	A	1640
114 IF ( IVDPTS .LE. 1 ) GO TO 116	A	1650
DO 115 I=2,IVDPTS	A	1660
IF ( VMASK(I-1) .GT. VMASK(I) ) CALL ERROR (10)	A	1670
115 CONTINUE	A	1680
C	A	1690
C EDIT VOID AND EXPOSURE MASKS	A	1700
116 CALL HEADER (1)	A	1710
WRITE (6,7000) ITSETS, IEXPTS, IVDPTS	A	1720
WRITE (6,7002) (IEX, BUMASK(IEX), IEX=1,IEXPTS)	A	1730
IF ( .NOT. NOVOID ) WRITE (6,7004) (IVD, VMASK(IVD), IVD=1,IVDPTS)	A	1740
C	A	1750
IF ( NOXE ) GO TO 3	A	1760
C READ XE + I FISSION YIELDS	A	1770
DO 5 NTS=1,ITSETS	A	1780
5 READ (5,*) (ITS,(YIX(ITS,IEX),IEX=1,IEXPTS))	A	1790
C	A	1800

C	READ CROSS SECTION INTERPOLATING TABLES	A	1810
3	DO 10 NTS=1,ITSETS	A	1820
	CALL HEADER (1)	A	1830
	READ (5,*) ITS	A	1840
	READ (5,8000) WORD	A	1850
	WRITE (6,120) ITS, WORD	A	1860
	IF ( NOVOID ) GO TO 6	A	1870
	DO 4 IVD=1,IVDPTS	A	1880
	WRITE (6,122) VMASK(IVD)	A	1890
	DO 4 IEX=1,IEXPTS	A	1900
	N = ITS+DIMTS*(DIMEX*(IVD-1)+IEX-1)	A	1910
	READ (5,*) DFI(N),SAFI(N),NSFFI(N),KSFFI(N),SDSI(N),	A	1920
	\$ DTI(N),SATI(N),NSFTI(N),KSFTI(N)	A	1930
	WRITE (6,124) BUMASK(IEX),DFI(N),SAFI(N),NSFFI(N),KSFFI(N),SDSI(N)	A	1940
	\$, DTI(N),SATI(N),NSFTI(N),KSFTI(N)	A	1950
	IF ( NOXE ) GO TO 4	A	1960
	READ (5,*) XEND, SAXE(N), SFF, SFT	A	1970
	SATI(N) = SATI(N) - XEND*SAXE(N)	A	1980
	PRSA1(N) = YIX(ITS,IEX)*SFF*SAXE(N)*1.E-24	A	1990
	PRSA2(N) = YIX(ITS,IEX)*SFT*SAXE(N)*1.E-24	A	2000
4	CONTINUE	A	2010
	GO TO 9	A	2020
6	WRITE (6,126)	A	2030
	DO 8 IEX=1,IEXPTS	A	2040
	N = ITS+(IEX-1)*DIMTS	A	2050
	READ (5,*) DFI(N),SAFI(N),NSFFI(N),KSFFI(N),SDSI(N),	A	2060
	\$ DTI(N),SATI(N),NSFTI(N),KSFTI(N)	A	2070
	WRITE (6,124) BUMASK(IEX),DFI(N),SAFI(N),NSFFI(N),KSFFI(N),SDSI(N)	A	2080
	\$, DTI(N),SATI(N),NSFTI(N),KSFTI(N)	A	2090
	IF ( NOXE ) GO TO 8	A	2100
	READ (5,*) XEND, SAXE(N), SFF, SFT	A	2110
	SATI(N) = SATI(N) - XEND*SAXE(N)	A	2120
	PRSA1(N) = YIX(ITS,IEX)*SFF*SAXE(N)*1.E-24	A	2130
	PRSA2(N) = YIX(ITS,IEX)*SFT*SAXE(N)*1.E-24	A	2140
8	CONTINUE	A	2150
9	IF ( NOXE ) GO TO 10	A	2160

CALL HEADER (1)	A	2170
WRITE (6,121) ITS,WORD	A	2180
DO 7 IVD=1,IVDPTS	A	2190
IF ( .NOT. NOVOID ) WRITE (6,122) VMASK(IVD)	A	2200
IF ( NOVOID ) WRITE (6,126)	A	2210
DO 7 IEX=1,IEXPTS	A	2220
N = ITS + DIMTS*(DIMEX*(IVD-1)+IEX-1)	A	2230
SFF = PRSA1(N)/(YIX(ITS,IEX)*SAXE(N))*1.E24	A	2240
SFT = PRSA2(N)/(YIX(ITS,IEX)*SAXE(N))*1.E24	A	2250
WRITE (6,129) BUMASK(IEX), SATI(N), SFF, SFT, SAXE(N)	A	2260
7 SAXE(N) = SAXE(N)*1.E-24	A	2270
10 CONTINUE	A	2280
C	A	2290
C	A	2300
IF ( NOXE ) GO TO 13	A	2310
C EDIT XE + I FISSION YIELDS	A	2320
CALL HEADER (1)	A	2330
WRITE (6,123) (I,I=1,ITSETS)	A	2340
WRITE (6,126)	A	2350
DO 19 IEX=1,IEXPTS	A	2360
19 WRITE (6,125) BUMASK(IEX),(YIX(ITS,IEX),ITS=1,ITSETS)	A	2370
CALCULATE FACTOR USED IN ABSOLUTE FLUX CALCULATION	A	2380
PD = MWTH*PCTPWR*6.2418E16/(DZ*DX*UX)	A	2390
IF ( QTRKOR ) PD = PD*.25	A	2400
C	A	2410
C READ TABLE SET ASSIGNMENTS	A	2420
13 IF ( ITSAS .EQ. 1 ) GO TO 650	A	2430
READ (5,*) FTYPE	A	2440
IF ( ITSAS .EQ. 3 ) GO TO 652	A	2450
DO 654 LB=1,NP	A	2460
LT = LB + NDSMNP	A	2470
DO 654 L=LB,LT,NP	A	2480
654 TABSET(L) = FTYPE(LB)	A	2490
GO TO 650	A	2500
C	A	2510
652 CALL HEADER (1)	A	2520



WRITE (6,670) (K,K=1,NZ)	A	2530
DO 658 NTS = 1,IFTYPS	A	2540
READ (5,*) IFT, (TSOVLY(IFT,K),K=1,N06)	A	2550
658 WRITE (6,662) IFT, (TSOVLY(IFT,K),K=1,N06)	A	2560
DO 660 LB=1,NP	A	2570
LT = LB + NDSMNP	A	2580
K = 0	A	2590
IFT = FTYPE(LB)	A	2600
DO 660 L=LB,LT,NP	A	2610
K = K + 1	A	2620
660 TABSET(L) = TSOVLY(IFT,K)	A	2630
650 CALL EDIT (7)	A	2640
C	A	2650
C READ CONTROL ROD POSITIONS	A	2660
IF ( NOCROD ) GO TO 11	A	2670
IF ( NBANKS .GT. 0 ) GO TO 623	A	2680
READ (5,*) NOTCH	A	2690
GO TO 621	A	2700
C DEFINE ROD BANKS	A	2710
623 CALL HEADER (1)	A	2720
WRITE (6,4034)	A	2730
IND = 0	A	2740
DO 608 NB=1,NBANKS	A	2750
READ (5,*) ID, NR, NOTCH	A	2760
M = 2*NR	A	2770
WRITE (6,4036) ID, (NOTCH(N),N=1,M)	A	2780
NRODS(ID,1) = NR	A	2790
NRODS(ID,2) = IND+1	A	2800
DO 608 N=1,NR	A	2810
IND = IND+1	A	2820
M = 2*N	A	2830
608 RODIND(IND) = LOC (NOTCH(M-1), NOTCH(M), 1)	A	2840
C SET ROD BANKS	A	2850
DO 612 L=1,NP	A	2860
612 NOTCH(L) = 0	A	2870
DO 610 NB=1,NBANKS	A	2880

READ (5,*) ID, NTCH	A	2890
610 CALL BANK (ID, NTCH)	A	2900
C	A	2910
C READ CONTROL ROD WORTHS	A	2920
621 CALL EDIT (6)	A	2930
DO 604 NTS=1,ICNTRL	A	2940
READ (5,*) ITS	A	2950
CALL HEADER (1)	A	2960
WRITE (6,6004) ITS	A	2970
IF ( NOVOID ) GO TO 602	A	2980
DO 600 IVD=1,IVDPTS	A	2990
WRITE (6,6000) VMASK(IVD)	A	3000
READ (5,*) (WORTH(ITS,IEX,IVD),IEX=1,IEXPTS)	A	3010
600 WRITE (6,6002) (BUMASK(IEX),WORTH(ITS,IEX,IVD),IEX=1,IEXPTS)	A	3020
GO TO 604	A	3030
602 READ (5,*) (WORTH(ITS,IEX,1),IEX=1,IEXPTS)	A	3040
WRITE (6,6002 ) (BUMASK(IEX),WORTH(ITS,IEX,1),IEX=1,IEXPTS)	A	3050
604 CONTINUE	A	3060
C	A	3070
C READ FUEL EXPOSURE ARRAY	A	3080
11 IF ( IEXRD) 12,18,16	A	3090
12 DO 14 LC=1,NP	A	3100
READ (5,*) I,J	A	3110
IF ( I .EQ. -99) GO TO 18	A	3120
LB = LOC (I,J,1)	A	3130
LT = LB+NDSMNP	A	3140
K = 0	A	3150
READ (5,*) ASMEX	A	3160
DO 14 L=LB,LT,NP	A	3170
K = K+1	A	3180
14 BU(L) = ASMEX(K)	A	3190
GO TO 18	A	3200
16 READ (5,*) BU	A	3210
C	A	3220
C READ INITIAL POWER GUESS	A	3230
18 IF ( IPDRD .NE. 1) GO TO 21	A	3240

READ (5,*) POWER	A	3250
CALL NORM (POWER,NODES)	A	3260
21 IF ( NOVOID ) GO TO 24	A	3270
C	A	3280
CALCULATE THERMAL-HYDRAULIC CONSTANTS FOR VOID AND MCHFR CALCULATIONS	A	3290
CALL HEADER (1)	A	3300
PMWTH = PCTPWR*.01*MWTH	A	3310
AVFLUX = AVHFLX*PEAKF*PCTPWR*.01	A	3320
XIN = (HIN-HSAT)/HFG	A	3330
AVPWR = PCTPWR*MWTH*3.4127E4/FLOAT(NODES)	A	3340
IF ( QTRKOR ) AVPWR = AVPWR*.25	A	3350
AC3 = VF/VG	A	3360
WRITE (6,4012) PMWTH,AVHFLX,PEAKF,VG,VF,HIN,HFG,HSAT,PSIA,TFLOW,	A	3370
\$ AREA,AC1,AC2	A	3380
C	A	3390
C READ FLOW CHANNEL CHARACTERISTICS	A	3400
IF ( IFLOW .EQ. 0) GO TO 23	A	3410
READ (5,*) CTYPE	A	3420
WRITE (6,4030)	A	3430
DO 20 N=1,IFLOW	A	3440
READ (5,*) M, B1(M), B2(M), B3(M)	A	3450
20 WRITE (6,4032) M, B1(M), B2(M), B3(M)	A	3460
CALL EDIT (9)	A	3470
23 CALL VOIDFR	A	3480
C	A	3490
CALCULATE NODAL WEIGHTING FACTORS	A	3500
24 R = DX*DX/DZ/DZ	A	3510
B1N = 3.*A1/(3.*A1+(1.-A1)*(R+2.))	A	3520
B2N = 3.*A2/(3.*A2+(1.-A2)*(R+2.))	A	3530
CF = B1N/6./A1*(1.-A1)	A	3540
CT = B2N/6./A2*(1.-A2)	A	3550
RCF = R*CF	A	3560
RCT = R*CT	A	3570
C	A	3580
CALCULATE GEOMETRIC CONSTANTS USED IN INNER ITERATION	A	3590
GX = 2./DX/DX	A	3600

GZ = 2./DZ/DZ	A 3610
C	A 3620
IF ( .NOT. NOHAL ) GO TO 300	A 3630
C	A 3640
C-----	A 3650
C	A 3660
C BURN TYPE PROBLEM	A 3670
C	A 3680
CALCULATE CORE AVERAGE EXPOSURE	A 3690
AVEXP = 0.	A 3700
DO 202 L=1,NODES	A 3710
202 AVEXP = AVEXP+BU(L)	A 3720
AVEXP = AVEXP/FLOAT(NODES)	A 3730
C READ BURNUP TIME STEP	A 3740
READ (5,*) BURN	A 3750
IF ( EOF(5) ) 208,800	A 3760
800 IF ( BURN .LT. 0.) BURN = -BURN/2400.*PCTPWR*MWTH/TONNES	A 3770
CALCULATE INITIAL POWER DISTRIBUTION	A 3780
CALL STG	A 3790
CALL EDIT (5)	A 3800
CALL EDIT (3)	A 3810
IF ( IEDIT(6) .EQ. -1 ) CALL EDIT (10)	A 3820
IF ( NOVOID ) GO TO 204	A 3830
CALL EDIT (1)	A 3840
CALL EDIT (4)	A 3850
CALL EDIT (2)	A 3860
CALCULATE NEW EXPOSURE ARRAY	A 3870
204 DO 206 L=1,NODES	A 3880
206 BU(L) = BU(L)+POWER(L)*BURN	A 3890
AVEXP = AVEXP+BURN	A 3900
CALCULATE NEW POWER DISTRIBUTION	A 3910
CALL STG	A 3920
CALL EDIT (5)	A 3930
CALL EDIT (3)	A 3940
IF ( IEDIT(6) .EQ. -1 ) CALL EDIT (10)	A 3950
IF ( IEDIT(3) .EQ. -1 ) CALL EDIT (8)	A 3960

IF ( NOVOID ) GO TO 210	A 3970
CALL EDIT (1)	A 3980
CALL EDIT (4)	A 3990
CALL EDIT (2)	A 4000
C	A 4010
C READ DATA FOR NEXT TIME STEP	A 4020
210 NEWTH = 0	A 4030
NEWF = 0	A 4040
NEWCR = 0	A 4050
NTSTEP = NTSTEP + 1	A 4060
READ (5,*) BURN, NEWTH, NEWF, NEWCR	A 4070
IF ( EOF(5) ) 234,802	A 4080
802 IF ( BURN .LT. 0.) BURN = -BURN/2400.*PCTPWR*MWTH/TONNES	A 4090
IF ( NEWTH .EQ. 0) GO TO 220	A 4100
C READ NEW THERMAL-HYDRAULIC DATA	A 4110
IFLOW = 0	A 4120
OLDFLO = TFLOW	A 4130
READ (5, INPUT)	A 4140
PMWTH = PCTPWR*.01*MWTH	A 4150
AVFLUX = AVHFLX*PEAKF*PCTPWR*.01	A 4160
XIN = (HIN-HSAT)/HFG	A 4170
AVPWR = PCTPWR*MWTH*3.4127E4/FLOAT (NODES)	A 4180
IF ( QTRKOR ) AVPWR = AVPWR*.25	A 4190
AC3 = VF/VG	A 4200
CALL HEADER (1)	A 4210
WRITE (6,4012) PMWTH,AVHFLX,PEAKF,VG,VF,HIN,HFG,HSAT,PSIA,TFLOW,	A 4220
\$ AREA,AC1,AC2	A 4230
IF ( IFLOW .EQ. 0) GO TO 220	A 4240
WRITE (6,4030)	A 4250
DO 212 N=1,IFLOW	A 4260
READ (5,*) M, B1(M), B2(M), B3(M)	A 4270
212 WRITE (6,4032) M, B1(M), B2(M), B3(M)	A 4280
CALL EDIT (9)	A 4290
220 IF ( NEWF .EQ. 0) GO TO 226	A 4300
C READ NEW FUEL TYPE ARRAY	A 4310
DO 2222 L=1,NODES	A 4320

2222	BU(L) = BU(L)+POWER(L)*BURN	A	4330
	CALL HEADER (1)	A	4340
	WRITE (6,2002) NEWF	A	4350
	DO 224 NN=1,NEWF	A	4360
	READ (5,*) IFT, ICTYP, I, J	A	4370
	WRITE (6,2004) I,J	A	4380
	LB = LOC (I,J,1)	A	4390
	READ (5,*) ASMEX	A	4400
	FTYPE(LB) = IFT	A	4410
	LT = LB + NDSMNP	A	4420
	IF ( ITSAS .EQ. 3 ) GO TO 696	A	4430
	DO 690 L=LB,LT,NP	A	4440
690	TABSET(L) = IFT	A	4450
	GO TO 699	A	4460
696	K = 0	A	4470
	DO 698 L=LB,LT,NP	A	4480
	K = K + 1	A	4490
698	TABSET(L) = TSOVLY(IFT,K)	A	4500
699	CTYPE(LB) = ICTYP	A	4510
	K = 0	A	4520
	LT = LB+NDSMNP	A	4530
	DO 224 L=LB,LT,NP	A	4540
	K = K+1	A	4550
224	BU(L) = ASMEX(K)	A	4560
C		A	4570
	IF ( NEWCR .EQ. 0 ) GO TO 206	A	4580
	IF ( NBANKS .EQ. 0 ) GO TO 226	A	4590
C	MOVE NEWCR ROD BANKS TO NEW POSITIONS	A	4600
	DO 280 I=1,NEWCR	A	4610
	READ (5,*) ID, NTCH	A	4620
280	CALL BANK (ID, NTCH)	A	4630
	GO TO 206	A	4640
226	IF ( NEWCR) 228,206,230	A	4650
C	READ NEW CONTROL ROD POSITION ARRAY	A	4660
C	IF NEWCR = -1 ... READ NEW NOTCH ARRAY	A	4670
C	0 ... NO CHANGE IN NOTCH ARRAY	A	4680

C	N ... READ N NEW NOTCH POSITIONS	A	4690
228	READ (5,*) NOTCH	A	4700
	CALL EDIT (6)	A	4710
	DO 2229 L=1,NODES	A	4720
2229	F(L) = 1.	A	4730
	GO TO 204	A	4740
230	DO 232 N=1,NEWCR	A	4750
	READ (5,*) I, J, NTCH	A	4760
	L = LOC (I,J,1)	A	4770
232	NOTCH(L) = NTCH	A	4780
	CALL EDIT (6)	A	4790
	GO TO 204	A	4800
C		A	4810
	CALCULATE POWER DISTRIBUTION FOR CASE WITH NO TIME STEP	A	4820
208	CALL STG	A	4830
	CALL EDIT (5)	A	4840
	CALL EDIT (3)	A	4850
	IF ( NOVOID ) GO TO 234	A	4860
	CALL EDIT (1)	A	4870
	CALL EDIT (4)	A	4880
	CALL EDIT (2)	A	4890
234	IF ( IEDIT(3) .EQ. 1 ) CALL EDIT (8)	A	4900
	IF ( IEDIT(6) .NE. 0 ) CALL EDIT (10)	A	4910
	WRITE (6,2000)	A	4920
	STOP	A	4930
C	-----	A	4940
C		A	4950
C	HALING SOLUTION	A	4960
C		A	4970
300	READ (5,*) BURN	A	4980
	IF ( BURN .LT. 0.) BURN = -BURN/2400.*PCTPWR*MWTH/TONNES	A	4990
	AVEXP = 0.	A	5000
	DO 301 L=1,NODES	A	5010
301	AVEXP = AVEXP + BU(L)	A	5020
	AVEXP = AVEXP/FLOAT(NODES)	A	5030
	CALL EDIT (3)	A	5040

CALL HEADER (1)	A	5050
CALL HALING (BURN)	A	5060
AVEXP = AVEXP + BURN	A	5070
CALL EDIT (3)	A	5080
CALL EDIT (5)	A	5090
IF ( IEDIT(3) .EQ. 1 ) CALL EDIT (8)	A	5100
IF ( IEDIT(6) .NE. 0 ) CALL EDIT (10)	A	5110
IF ( NOVOID ) GO TO 316	A	5120
CALL EDIT (4)	A	5130
CALL EDIT (2)	A	5140
CALL EDIT (1)	A	5150
316 WRITE (6,2000)	A	5160
STOP	A	5170
C	A	5180
C-----	A	5190
C	A	5200
7004 FORMAT (//10X,'* VOID MASK *//10X,'TABLE'/	A	5210
\$10X,'INDEX VOID'/10X,'===== '/10(/I15,F11.3))	A	5220
6004 FORMAT (1X,'*** CONTROL ROD WORTH FOR TABLE SET',I5,' ***',//	A	5230
\$20X,'EXPOSURE'/20X,'(MWD/MT) WORTH'/20X,'===== '	A	5240
6000 FORMAT (4X,'VOID =' ,F8.2)	A	5250
6002 FORMAT (18X,E10.2,F8.2)	A	5260
7000 FORMAT (1X,I3,' CROSS SECTION INTERPOLATING TABLES WILL BE READ AT	A	5270
\$' ,I3,' EXPOSURE AND',I3,' ' VOID INTE	A	5280
\$RPOLATING POINTS'///10X,'* EXPOSURE MASK *///10X,'TABLE EXPOS	A	5290
\$URE'/10X,'INDEX (MWD/MT)'/10X,'===== '	A	5300
7002 FORMAT (10X,I5,F13.0)	A	5310
2000 FORMAT (//1X,'*** END OF CASE ***')	A	5320
2002 FORMAT (1X,I5,' NEW ASSEMBLIES LOADED AT THESE LOCATIONS'//21X,'	A	5330
\$ I J'/21X,'===== '	A	5340
2004 FORMAT (20X,2I6)	A	5350
1000 FORMAT (20A4)	A	5360
4010 FORMAT (3X,'NODE WIDTH =' ,F10.3,' CM.'/3X,'NODE HEIGHT =' ,F10.3,'	A	5370
\$ CM.'/3X,'NODAL WEIGHTING FACTORS: A1 =' ,F10.4/30X,'A2 =' ,	A	5380
\$F10.4)	A	5390
4000 FORMAT (/9X,'===== CONVERGENCE PARAMETERS =====')	A	5400



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$//10X, A 5410
$! OUTER ITERATION WILL BE CONVERGED TO ',E10.2/10X, A 5420
$! POWER ITERATION WILL BE CONVERGED TO ',E10.2//10X, A 5430
$! MAXIMUM NUMBER OF INNER ITERATIONS =',15/10X, A 5440
$! MAXIMUM NUMBER OF OUTER ITERATIONS =',15/10X, A 5450
$! MAXIMUM NUMBER OF POWER ITERATIONS =',15//) A 5460
8000 FORMAT (20A4) A 5470
120 FORMAT (1X, '===== CROS A 5480
$S SECTION INTERPOLATING TABLE ===== A 5490
$===== '/1X, 'TABLE SET', I3, 5X, 20A4//27X, '----- A 5500
$--- F A S T ----- T H E R M A 5510
$ A L ----- '/13X, 'EXPOSURE', 35X, 'NU-', 6X, 'KAPPA-', 8X, 'DOWN A 5520
$', 35X, 'NU-', 6X, 'KAPPA-', '/13X, '(MWD/MT)', 13X, 'D ABSORBTION FISS A 5530
$! ON FISSION SCATTERING', 13X, 'D ABSORBTION FISSION FI A 5540
$SSION', '/11X, '=====', 2X, 5(' ====='), 2X, 4(' =====') A 5550
122 FORMAT (1X, 'VOID =', F5.2) A 5560
124 FORMAT (11X, F10.0, 2X, 5E12.4, 2X, 4E12.4) A 5570
126 FORMAT (/) A 5580
121 FORMAT (1X, '===== CROS A 5590
$S SECTION INTERPOLATING TABLE ===== A 5600
$===== '/1X, 'TABLE SET', I3, 5X, 20A4//28X, 'THERMAL', 30X, 'XE-1 A 5610
$35'/13X, 'EXPOSURE ABSORBTION FAST THERMAL MICR A 5620
$0'/13X, '(MWD/MT) XE REMOVED FISSION FISSION ABSORBTION A 5630
$'/11X, '=====', 2X, 4(' =====') A 5640
123 FORMAT (1X, '===== XE-135 + I-135 FISSION YIELDS ===== '/3X, 'Ex A 5650
$POSURE', /3X, '(MWD/MT) TABLE SET =', 15I8) A 5660
125 FORMAT (1X, F10.0, 13X, 15F8.4) A 5670
129 FORMAT (11X, F10.0, 2X, 3E12.4, F12.0) A 5680
4012 FORMAT (15X, '* THERMAL-HYDRAULIC DATA FOR THIS TIME STEP *'/// A 5690
$15X, 'CORE POWER', T37, F12.1, T52, 'MW-THERMAL'// A 5700
$15X, 'AV. HEAT FLUX', T37, F12.0, T52, 'BTU/HR/FT**2'// A 5710
$15X, 'PEAKING FACTOR', T37, F12.2// A 5720
$15X, 'SAT. VAP. SPEC. VOL.', T37, F12.4, T52, 'FT**3/LBM'// A 5730
$15X, 'SAT. LIQ. SPEC. VOL.', T37, F12.5, T52, 'FT**3/LBM'// A 5740
$15X, 'INLET ENTHALPY', T37, F12.2, T52, 'BTU/LBM'// A 5750
$15X, 'HEAT OF VAPORIZATION', T37, F12.2, T52, 'BTU/LBM'// A 5760

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$15X,'ENTHALPY OF SAT. LIQ.',T37,F12.2,T52,'BTU/LBM'// A 5770
$15X,'CORE PRESSURE',T37,F12.1,T52,'PSIA'// A 5780
$15X,'INCORE FLOW',T37,IPE12.3,T52,'LBM/HR'// A 5790
$15X,'FLOW AREA PER CHANNEL',T37,OPF12.4,T52,'FT**2'// A 5800
$15X,'ARMAND CORRELATION COEFICIENTS'/25X,'AC1 =',F8.3/25X,'AC2 =', A 5810
$F8.3) A 5820
4020 FORMAT(/9X,'===== BOUNDARY CONDITIONS =====' A 5830
$====='//11X,'ALB = A + B*VOID(L)'/11X,'ALB = 1. FOR ZERO CURREN A 5840
$T'/18X,'0. FOR ZERO FLUX'/17X,'XX. FOR LEAKAGE SPECIFIED BY A A 5850
$AND B' A 5860
$//32X,'--- FAST ---',12X,'- THERMAL -'/33X,'A',8X,'B',14X, A 5870
$'A',8X,'B'/21X,2(' ===== ')/16X,'FACE #1',4X,2F9 A 5880
$.2,6X,2F9.2/16X,'FACE #2',4X,2F9.2,6X,2F9.2/16X,'FACE #3',4X,2F9 A 5890
$.2,6X,2F9.2/16X,'FACE #4',4X,2F9.2,6X,2F9.2/17X,'BOTTOM',4X,2F9. A 5900
$2,6X,2F9.2/20X,'TOP',4X,2F9.2,6X,2F9.2) A 5910
4022 FORMAT(/9X,'===== BOUNDARY CONDITIONS =====' A 5920
$====='//11X,'ALB = A + B*VOID(L)'/11X,'ALB = 1. FOR ZERO CURREN A 5930
$T'/18X,'0. FOR ZERO FLUX'/17X,'XX. FOR LEAKAGE SPECIFIED BY A A 5940
$AND B' A 5950
$//32X,'--- FAST ---',12X,'- THERMAL -'/33X,'A',8X,'B',14X, A 5960
$'A',8X,'B'/21X,2(' ===== ')/16X,'FACE #1',4X,2F9 A 5970
$//17X,'BOTTOM',4X,2F9 A 5980
$.2,6X,2F9.2/20X,'TOP',4X,2F9.2,6X,2F9.2//9X,'QTR. CORE EDGE'/13X A 5990
$, 'ROW NUMBER'/20(I23,4X,2F9.2,6X,2F9.2//) A 6000
4030 FORMAT (/15X,'COOLANT FLOW VARIATION WITH POWER'/20X,'P = RELATI A 6010
$VE POWER IN CHANNEL'/20X,'FLOW = RELATIVE COOLANT FLOW IN CHANNEL A 6020
$ (UNNORMALIZED)'/20X,'CHANNEL TYPE'/20X,'=====')// A 6030
4032 FORMAT (2X,I30,5X,'FLOW =',E10.3,' +',E10.3,' *P +',E10.3,' *P**2 A 6040
$ '//) A 6050
4034 FORMAT (10X,'*** ROD BANK ASSIGNMENT ***'//) A 6060
4036 FORMAT (20X,'BANK NUMBER',I5/20X,'LOCATION OF NODES WHICH ARE RODD A 6070
$ED BY THIS BANK: (I,J)'/25X,8('(',I3,',',I3,')',4X)'/) A 6080
670 FORMAT (1X,'TABLE SET TO FUEL TYPE ASSIGNMENT'/1X,'FUEL TABLE A 6090
$SET'/1X,'TYPE K=',N06I4,/1X,'---- ',N06(----)'/) A 6100
662 FORMAT (1X,I4,4X,N06I4) A 6110
C A 6120

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END

A 6130

C#=====	B	10
C=====	B	20
C	B	30
C	B	40
SUBROUTINE INTERP	B	50
C	B	60
C-----	B	70
C THIS ROUTINE INTERPOLATES USING THE INPUT CROSS SECTION	B	80
C INTERPOLATING TABLES TO DETERMINE NODAL CONSTANTS	B	90
C-----	B	100
C	B	110
COMMON /HEADR / NTITLE(20), AVEXP, NTSTEP	B	120
COMMON /CSINP / BUMASK(N07), VMASK(N08), DFI(N10), DTI(N10)	B	130
\$            SAFI(N10), SATI(N10), NSFFI(N10), NSFTI(N10),	B	140
\$            SDSI(N10), IEXPTS, IVDPTS, ITSETS,	B	150
\$            KSFTI(N10), KSFFI(N10),	B	160
\$            DIMTS, DIMEX, DIMVD, NOTCH(N02),	B	170
\$            TABSET(N01), WORTH(N10)	B	180
INTEGER DIMTS, DIMEX, DIMVD, TABSET	B	190
REAL NSFFI, NSFTI, KSFFI, KSFTI	B	200
COMMON /FUEL / FTYPE(N02), BU(N01), VOID(N01)	B	210
INTEGER FTYPE	B	220
COMMON /NODAL / DF (N01), DT (N01), KSFF (N01)	B	230
\$,            KSFT (N01), TFR (N01), NSF1G (N01)	B	240
\$,            C2 (N01), C4 (N01), C6 (N03)	B	250
\$,            C7 (N01)	B	260
REAL KSFF, KSFT, NSF1G	B	270
COMMON /GEOM / NODES, NP, NDSMNP, NX	B	280
\$,            NZ, NXM1, GX, GZ	B	290
\$,            NCOL (N04)	B	300
COMMON /BC / AF1, AF3, AF5, AF6, AFQ(N04),	B	310
\$            BF1, BF3, BF5, BF6, BFQ(N04),	B	320
\$            AT1, AT3, AT5, AT6, ATQ(N04),	B	330
\$            BT1, BT3, BT5, BT6, BTQ(N04)	B	340
EQUIVALENCE (AFQ(1), AF2), (AFQ(2), AF4),	B	350
\$            (BFQ(1), BF2), (BFQ(2), BF4),	B	360

\$	(ATQ(1), AT2), (ATQ(2), AT4),	B	370
\$	(BTQ(1), BT2), (BTQ(2), BT4)	B	380
	COMMON /PROB / INTTYP, QTRKOR, NOHAL, NOCROD, NOVOID, IEDIT(6)	B	390
\$	NOXE, FIRST	B	400
	LOGICAL QTRKOR, NOHAL, NOCROD, NOVOID, NOXE, FIRST	B	410
	COMMON /FLUX / POWER(N01), FASTA(N01), THERMA(N01), F(N01)	B	420
	REAL NSFF, NSFT	B	430
	COMMON /XENON / PD, PRSA1(N10), PRSA2(N10), SAXE(N10)	B	440
	LOGICAL NOXEN	B	450
C		B	460
C	-----	B	470
C		B	480
C	INTTYP = 1 FOR NO EXPOSURE OR VOID INTERPOLATION	B	490
C	2 FOR EXPOSURE INTERPOLATION ONLY	B	500
C	3 FOR VOID INTERPOLATION ONLY	B	510
C	4 FOR EXPOSURE AND VOID INTERPOLATION	B	520
C		B	530
C	DF = FAST DIFFUSION COEFFICIENT	B	540
C	SAF = FAST ABSORBTION CROSS SECTION	B	550
C	NSFF = FAST NU-FISSION CROSS SECTION	B	560
C	KSFF = FAST KAPPA-FISSION CROSS SECTION	B	570
C	SDS = FAST TO THERMAL DOWN SCATTERING CROSS SECTION	B	580
C	DT = THERMAL DIFFUSION COEFFICIENT	B	590
C	SAT = THERMAL ABSORBTION CROSS SECTION	B	600
C	NSFT = THERMAL NU-FISSION CROSS SECTION	B	610
C	KSFT = THERMAL KAPPA-FISSION CROSS SECTION	B	620
C	SAX = XENON THERMAL MICRO-ABSORBTION CROSS SECTION	B	630
C	PR1 = (FAST FISSION C.S.) * (I-135 + XE-135 FISSION YIELD) *	B	640
C	(XENON MICRO-ABS. C.S.)	B	650
C	PR2 = SAME AS ABOVE FOR THERMAL GROUP	B	660
C	SA = THERMAL ABSORBTION CROSS SECTION WITH XE-135 REMOVED	B	670
C	TFR = ASSYMPTOTIC THERMAL TO FAST FLUX RATIO	B	680
C	NSF1G = ONE GROUP (FAST) NU-FISSION CROSS SECTION	B	690
C	RHO = CONTROL ROD WORTH	B	700
C		B	710
C	-----	B	720

C	NOXEN = NOXE	B	730
	IF ( FIRST ) NOXEN = .TRUE.	B	740
	IF ( NOXEN ) GO TO 3	B	750
	CALCULATE ABSOLUTE FLUXES	B	760
	SUM = 0.	B	770
	DO 1 L=1,NODES	B	780
1	SUM = SUM + FASTA(L)*KSFF(L) + THERMA(L)*KSFT(L)	B	790
	FACTOR = PD/SUM	B	800
	DO 2 L=1,NODES	B	810
	FASTA(L) = FACTOR*FASTA(L)	B	820
2	THERMA(L) = FACTOR*THERMA(L)	B	830
C		B	840
3	GO TO (10, 20, 30, 40), INTTYP	B	850
C=====		B	860
C		B	870
C		B	880
C		B	890
C	NO VOID OR EXPOSURE INTERPOLATION	B	900
10	DO 12 LB=1,NP	B	910
	LT = LB+NDSMNP	B	920
	DO 12 L=LB,LT,NP	B	930
	NTS = TABSET(L)	B	940
	DF (L) = DFI (NTS)	B	950
	DT (L) = DTI (NTS)	B	960
	SDS = SDSI (NTS)	B	970
	NSFF = NSFFI(NTS)	B	980
	NSFT = NSFTI(NTS)	B	990
	KSFF (L) = KSFFI(NTS)	B	1000
	KSFT (L) = KSFTI(NTS)	B	1010
	IF ( NOXEN ) GO TO 11	B	1020
C	XENON CORRECTION	B	1030
	PS1 = PRSA1(NTS)	B	1040
	PS2 = PRSA2(NTS)	B	1050
	SAX = SAXE (NTS)	B	1060
	SA = SATI(NTS)	B	1070
	SAT = SA + (PS1*FASTA(L) + PS2*THERMA(L))/(2.0997E-5 +	B	1080

\$	THERMA(L)*SAX)	B	1090
	GO TO 15	B	1100
11	SAT = SATI (NTS)	B	1110
15	SAF = SAFI (NTS)	B	1120
	C7 (L) = SAF+SDS	B	1130
	TFR (L) = SDS/SAT	B	1140
	NSF1G(L) = NSFF+TFR(L)*NSFT	B	1150
CHECK	TO SEE IF NODE IS RODDED	B	1160
	IF ( NOCROD ) GO TO 12	B	1170
	K = (L-1)/NP+1	B	1180
	IF ( NOTCH(LB) .LT. K) GO TO 12	B	1190
CHANGE	THERMAL ABSORBTION CROSS SECTION TO ACCOUNT FOR CONTROL ROD	B	1200
	RHO = WORTH(NTS)	B	1210
	SAT = NSFT*SDS*SAT/(NSFT*SDS-RHO*NSF1G(L)*SAT)	B	1220
	TFR(L) = SDS/SAT	B	1230
	NSF1G(L) = NSFF+TFR(L)*NSFT	B	1240
12	CONTINUE	B	1250
	GO TO 60	B	1260
C=====		B	1270
C		B	1280
C		B	1290
C	EXPOSURE INTERPOLATION ONLY	B	1300
20	DO 26 LB=1,NP	B	1310
	LT = LB+NDSMNP	B	1320
	DO 26 L=LB,LT,NP	B	1330
C	LOCATE EXPOSURE INTERPOLATING POINTS	B	1340
	E = BU(L)	B	1350
	DO 22 NE = 2,IEXPTS	B	1360
	IF ( E .LT. BUMASK(NE)) GO TO 24	B	1370
22	CONTINUE	B	1380
	NE = IEXPTS	B	1390
CALCULATE	INTERPOLATING WEIGHTS AND INDICES	B	1400
24	W2 = (E-BUMASK(NE-1))/(BUMASK(NE)-BUMASK(NE-1))	B	1410
	W1 = 1.-W2	B	1420
	M2 = TABSET(L)+(NE-1)*DIMTS	B	1430
	M1 = M2-DIMTS	B	1440

C		B	1450
	DF (L) = W1*DFI (M1) + W2*DFI (M2)	B	1460
	DT (L) = W1*DTI (M1) + W2*DTI (M2)	B	1470
	SDS = W1*SDSI (M1) + W2*SDSI (M2)	B	1480
	NSFF = W1*NSFFI (M1) + W2*NSFFI (M2)	B	1490
	NSFT = W1*NSFTI (M1) + W2*NSFTI (M2)	B	1500
	KSFF (L) = W1*KSFFI (M1) + W2*KSFFI (M2)	B	1510
	KSFT (L) = W1*KSFTI (M1) + W2*KSFTI (M2)	B	1520
	IF ( NOXEN ) GO TO 21	B	1530
C	XENON CORRECTION	B	1540
	PS1 = W1*PRSA1 (M1) + W2*PRSA1 (M2)	B	1550
	PS2 = W1*PRSA2 (M1) + W2*PRSA2 (M2)	B	1560
	SAX = W1*SAXE (M1) + W2*SAXE (M2)	B	1570
	SA = W1*SATI (M1) + W2*SATI (M2)	B	1580
	SAT = SA + (PS1*FASTA(L) + PS2*THERMA(L))/(2.0997E-5 +	B	1590
	\$ THERMA(L)*SAX)	B	1600
	GO TO 25	B	1610
21	SAT = W1*SATI (M1) + W2*SATI (M2)	B	1620
25	SAF = W1*SAFI (M1) + W2*SAFI (M2)	B	1630
	C7 (L) = SAF+SDS	B	1640
	TFR (L) = SDS/SAT	B	1650
	NSF1G(L) = NSFF+TFR(L)*NSFT	B	1660
CHECK	TO SEE IF NODE IS RODDED	B	1670
	IF ( NOCRD ) GO TO 26	B	1680
	K = (L-1)/NP+1	B	1690
	IF ( NOTCH(LB) .LT. K) GO TO 26	B	1700
CHANGE	THERMAL ABSORPTION CROSS SECTION TO ACCOUNT FOR CONTROL ROD	B	1710
	RHO = W1*WORTH(M1) + W2*WORTH(M2)	B	1720
	SAT = NSFT*SDS*SAT/(NSFT*SDS-RHO*NSF1G(L)*SAT)	B	1730
	TFR(L) = SDS/SAT	B	1740
	NSF1G(L) = NSFF+TFR(L)*NSFT	B	1750
26	CONTINUE	B	1760
	GO TO 60	B	1770
C=====		B	1780
C		B	1790
C		B	1800



C VOID INTERPOLATION ONLY	B	1810
30 DO 36 LB=1,NP	B	1820
LT = LB+NDSMNP	B	1830
DO 36 L=LB,LT,NP	B	1840
C LOCATE VOID INTERPOLATING POINTS	B	1850
V = VOID(L)	B	1860
DO 32 NV=2,IVDPTS	B	1870
IF ( V .LT. VMASK(NV)) GO TO 34	B	1880
32 CONTINUE	B	1890
NV = IVDPTS	B	1900
CALCULATE INTERPOLATING WEIGHTS AND INDICES	B	1910
34 W2 = (V-VMASK(NV-1))/(VMASK(NV)-VMASK(NV-1))	B	1920
W1 = 1.-W2	B	1930
M2 = TABSET(L)+(NV-1)*DIMTS*DIME X	B	1940
M1 = M2-DIMTS*DIME X	B	1950
C	B	1960
DF (L) = W1*DFI (M1) + W2*DFI (M2)	B	1970
DT (L) = W1*DTI (M1) + W2*DTI (M2)	B	1980
SDS = W1*SDSI (M1) + W2*SDSI (M2)	B	1990
NSFF = W1*NSFFI(M1) + W2*NSFFI(M2)	B	2000
NSFT = W1*NSFTI(M1) + W2*NSFTI(M2)	B	2010
KSFF (L) = W1*KSFFI(M1) + W2*KSFFI(M2)	B	2020
KSFT (L) = W1*KSFTI(M1) + W2*KSFTI(M2)	B	2030
IF ( NOXEN ) GO TO 31	B	2040
C XENON CORRECTION	B	2050
PS1 = W1*PRSA1(M1) + W2*PRSA1(M2)	B	2060
PS2 = W1*PRSA2(M1) + W2*PRSA2(M2)	B	2070
SAX = W1*SAXE (M1) + W2*SAXE (M2)	B	2080
SA = W1*SATI (M1) + W2*SATI (M2)	B	2090
SAT = SA + (PS1*FASTA(L) + PS2*THERMA(L))/(2.0997E-5 +	B	2100
\$ THERMA(L)*SAX)	B	2110
GO TO 35	B	2120
31 SAT = W1*SATI (M1) + W2*SATI (M2)	B	2130
35 SAF = W1*SAFI (M1) + W2*SAFI (M2)	B	2140
C7 (L) = SAF+SDS	B	2150
TFR (L) = SDS/SAT	B	2160

NSF1G(L) = NSFF+TFR(L)*NSFT	B	2170
CHECK TO SEE IF NODE IS RODDED	B	2180
IF ( NOCROD ) GO TO 36	B	2190
K = (L-1)/NP+1	B	2200
IF ( NOTCH(LB) .LT. K) GO TO 36	B	2210
CHANGE THERMAL ABSORPTION CROSS SECTION TO ACCOUNT FOR CONTROL ROD	B	2220
RHO = W1*WORTH(M1) + W2*WORTH(M2)	B	2230
SAT = NSFT*SDS*SAT/(NSFT*SDS-RHO*NSF1G(L)*SAT)	B	2240
TFR(L) = SDS/SAT	B	2250
NSF1G(L) = NSFF+TFR(L)*NSFT	B	2260
36 CONTINUE	B	2270
GO TO 60	B	2280
C=====	B	2290
C	B	2300
C	B	2310
C EXPOSURE AND VOID INTERPOLATION	B	2320
40 DO 50 LB=1,NP	B	2330
LT = LB+NDSMNP	B	2340
DO 50 L=LB,LT,NP	B	2350
C LOCATE EXPOSURE INTERPOLATING POINTS	B	2360
E = BU(L)	B	2370
DO 42 NE = 2,IEXPTS	B	2380
IF ( E .LT. BUMASK(NE)) GO TO 44	B	2390
42 CONTINUE	B	2400
NE = IEXPTS	B	2410
C LOCATE VOID INTERPOLATING POINTS	B	2420
44 V = VOID(L)	B	2430
DO 46 NV=2,IVDPTS	B	2440
IF ( V .LT. VMASK(NV)) GO TO 48	B	2450
46 CONTINUE	B	2460
NV = IVDPTS	B	2470
CALCULATE INTERPOLATING WEIGHTS AND INDICES	B	2480
48 WE = (E-BUMASK (NE-1))/(BUMASK (NE)-BUMASK (NE-1))	B	2490
WV = (V-VMASK(NV-1))/(VMASK(NV)-VMASK(NV-1))	B	2500
W4 = WE*WV	B	2510
W3 = WE-W4	B	2520

W2 = WV-W4	B	2530
W1 = 1.-W2-WE	B	2540
M4 = TABSET(L)+DIMTS*((NE-1)+DIMEX*(NV-1))	B	2550
M3 = M4-DIMTS*DIMEX	B	2560
M2 = M4-DIMTS	B	2570
M1 = M3-DIMTS	B	2580
	B	2590
C	B	2600
DF (L) = W1*DFI (M1) + W2*DFI (M2) +	B	2610
\$ W3*DFI (M3) + W4*DFI (M4)	B	2620
DT (L) = W1*DTI (M1) + W2*DTI (M2) +	B	2630
\$ W3*DTI (M3) + W4*DTI (M4)	B	2640
SDS = W1*SDSI (M1) + W2*SDSI (M2) +	B	2650
\$ W3*SDSI (M3) + W4*SDSI (M4)	B	2660
NSFF = W1*NSFFI (M1) + W2*NSFFI (M2) +	B	2670
\$ W3*NSFFI (M3) + W4*NSFFI (M4)	B	2680
NSFT = W1*NSFTI (M1) + W2*NSFTI (M2) +	B	2690
\$ W3*NSFTI (M3) + W4*NSFTI (M4)	B	2700
KSFF (L) = W1*KSFFI (M1) + W2*KSFFI (M2) +	B	2710
\$ W3*KSFFI (M3) + W4*KSFFI (M4)	B	2720
KSFT (L) = W1*KSFTI (M1) + W2*KSFTI (M2) +	B	2730
\$ W3*KSFTI (M3) + W4*KSFTI (M4)	B	2740
IF ( NOXEN ) GO TO 41	B	2750
C XENON CORRECTION	B	2760
PS1 = W1*PRSA1 (M1) + W2*PRSA1 (M2) +	B	2770
\$ W3*PRSA1 (M3) + W4*PRSA1 (M4)	B	2780
PS2 = W1*PRSA2 (M1) + W2*PRSA2 (M2) +	B	2790
\$ W3*PRSA2 (M3) + W4*PRSA2 (M4)	B	2800
SAX = W1*SAXE (M1) + W2*SAXE (M2) +	B	2810
\$ W3*SAXE (M3) + W4*SAXE (M4)	B	2820
SA = W1*SATI (M1) + W2*SATI (M2) +	B	2830
\$ W3*SATI (M3) + W4*SATI (M4)	B	2840
SAT = SA + (PS1*FASTA(L) + PS2*THERMA(L))/(2.0997E-5 +	B	2850
\$ THERMA(L)*SAX)	B	2860
GO TO 45	B	2870
41 SAT = W1*SATI (M1) + W2*SATI (M2) +	B	2880
\$ W3*SATI (M3) + W4*SATI (M4)	B	

45	SAF	= W1*SAFI (M1) + W2*SAFI (M2) +	B	2890
	\$	W3*SAFI (M3) + W4*SAFI (M4)	B	2900
	C7	(L) = SAF+SDS	B	2910
	TFR	(L) = SDS/SAT	B	2920
	NSFIG(L)	= NSFF+TFR(L)*NSFT	B	2930
	CHECK	TO SEE IF NODE IS RODDED	B	2940
	IF	( NOCROD ) GO TO 50	B	2950
	K	= (L-1)/NP+1	B	2960
	IF	( NOTCH(LB) .LT. K) GO TO 50	B	2970
	CHANGE	THERMAL ABSORPTION CROSS SECTION TO ACCOUNT FOR CONTROL ROD	B	2980
	RHO	= W1*WORTH(M1) + W2*WORTH(M2) +	B	2990
	\$	W3*WORTH(M3) + W4*WORTH(M4)	B	3000
	SAT	= NSFT*SDS*SAT/(NSFT*SDS-RHO*NSFIG(L)*SAT)	B	3010
	TFR(L)	= SDS/SAT	B	3020
	NSFIG(L)	= NSFF+TFR(L)*NSFT	B	3030
50	CONTINUE		B	3040
C			B	3050
C	=====		B	3060
C			B	3070
C			B	3080
C	ADD B.C. CONTRIBUTIONS TO C7		B	3090
C	FACES 5 & 6		B	3100
60	DO 62 L=1,NP		B	3110
	C7(L)	= C7(L)-GZ*DF(L)*(AF5+BF5*VOID(L)-1.)	B	3120
	LL	= L+NDSMNP	B	3130
62	C7(LL)	= C7(LL)-GZ*DF(LL)*(AF6+BF6*VOID(LL)-1.)	B	3140
C	FACE 1		B	3150
	DO 64 LB=1,NX		B	3160
	LT	= LB+NDSMNP	B	3170
	DO 64 L=LB,LT,NP		B	3180
64	C7(L)	= C7(L)-GX*DF(L)*(AF1+BF1*VOID(L)-1.)	B	3190
C	FACE 3		B	3200
	LB	= 1	B	3210
	DO 68 I=1,NX		B	3220
	LT	= LB+NDSMNP	B	3230
	DO 66 L=LB,LT,NP		B	3240

66	C7(L) = C7(L)-GX*DF(L)*(AF3+BF3*VOID(L)-1.)	B	3250
68	LB = LB+NCOL(I)	B	3260
C		B	3270
	IF ( QTRKOR ) GO TO 76	B	3280
C		B	3290
C	-----	B	3300
C	RECTANGULAR PLANE GEOMETRY	B	3310
C	-----	B	3320
C		B	3330
C	FACE 2	B	3340
	LB1 = NP-NXM1	B	3350
	DO 72 LB=LB1,NP	B	3360
	LT = LB+NDSMNP	B	3370
	DO 72 L=LB,LT,NP	B	3380
72	C7(L) = C7(L)-GX*DF(L)*(AF2+BF2*VOID(L)-1.)	B	3390
C	-----	B	3400
C		B	3410
C	FACE 4	B	3420
	DO 74 LB=NX,NP,NX	B	3430
	LT = LB+NDSMNP	B	3440
	DO 74 L=LB,LT,NP	B	3450
74	C7(L) = C7(L)-GX*DF(L)*(AF4+BF4*VOID(L)-1.)	B	3460
	GO TO 131	B	3470
C		B	3480
C	-----	B	3490
C		B	3500
C	QUARTER CORE PLANE GEOMETRY	B	3510
C	-----	B	3520
C		B	3530
C	FACE 2	B	3540
76	J = 0	B	3550
	L1 = NP-NCOL(NX)+1	B	3560
	L2 = NP	B	3570
	DO 78 LB=L1,NP	B	3580
	LT = LB+NDSMNP	B	3590
	J = J+1	B	3600

	AF = AFQ(J)-1.	B	3610
	BF = BFQ(J)	B	3620
	DO 78 L=LB,LT,NP	B	3630
78	C7(L) = C7(L)-GX*DF(L)*(AF+BF*VOID(L))	B	3640
	I = NX	B	3650
	NCL = NCOL(NX)	B	3660
	DO 82 IC=2,NX	B	3670
	I = I-1	B	3680
	NCI = NCOL(I)	B	3690
	L2 = L2-NCL	B	3700
	L1 = L2-NCI+NCL+1	B	3710
	IF ( NCI .EQ. NCL) GO TO 82	B	3720
	DO 80 LB=L1,L2	B	3730
	LT = LB+NDSMNP	B	3740
	J = J+1	B	3750
	AF = AFQ(J)-1.	B	3760
	BF = BFQ(J)	B	3770
	DO 80 L=LB,LT,NP	B	3780
80	C7(L) = C7(L)-GX*DF(L)*(AF+BF*VOID(L))	B	3790
82	NCL = NCI	B	3800
C		B	3810
C FACE 4		B	3820
	LB = 0	B	3830
	DO 84 I=1,NX	B	3840
	LB = LB+NCOL(I)	B	3850
	LT = LB+NDSMNP	B	3860
	AF = AFQ(I)-1.	B	3870
	BF = BFQ(I)	B	3880
	DO 84 L=LB,LT,NP	B	3890
84	C7(L) = C7(L)-GX*DF(L)*(AF+BF*VOID(L))	B	3900
C-----		B	3910
C		B	3920
CALCULATE C5 & C6		B	3930
131	DO 132 L=1,NDSMNP	B	3940
	LT = L+NP	B	3950
	C6(L) = GZ*DF(L)*DF(LT)/(DF(L)+DF(LT))	B	3960

	C7(L) = C7(L)+C6(L)	B	3970
132	C7(LT) = C7(LT)+C6(L)	B	3980
C	-----	B	3990
C		B	4000
	CALCULATE C3 & C4	B	4010
	LR = -1	B	4020
	DO 138 I=1,NX	B	4030
	LL = LR+2	B	4040
	LR = LR+NCOL(I)	B	4050
	IF ( 1-NCOL(I)) 134,138,138	B	4060
134	DO 136 LB=LL,LR	B	4070
	LT = LB+NDSMNP	B	4080
	DO 136 L=LB,LT,NP	B	4090
	LP = L+1	B	4100
	C4(L) = GX*DF(L)*DF(LP)/(DF(L)+DF(LP))	B	4110
	C7(L) = C7(L)+C4(L)	B	4120
136	C7(LP) = C7(LP)+C4(L)	B	4130
138	CONTINUE	B	4140
C	-----	B	4150
C		B	4160
	CALCULATE C1 & C2	B	4170
	LL = 1	B	4180
	NCOLI = 0	B	4190
	DO 140 I=1,NXM1	B	4200
	LL = LL+NCOLI	B	4210
	NCOLI = NCOL(I)	B	4220
	LR = LL+NCOL(I+1)-1	B	4230
	DO 140 LB=LL,LR	B	4240
	LT = LB+NDSMNP	B	4250
	DO 140 L=LB,LT,NP	B	4260
	LP = L+NCOLI	B	4270
	C2(L) = GX*DF(L)*DF(LP)/(DF(L)+DF(LP))	B	4280
	C7(L) = C7(L)+C2(L)	B	4290
140	C7(LP) = C7(LP)+C2(L)	B	4300
C		B	4310
	RETURN	B	4320

END

B 4330



C*	=====	C	10
C	=====	C	20
C		C	30
C		C	40
	SUBROUTINE STG	C	50
C		C	60
C	-----	C	70
C	ROUTINE TO CALCULATE POWER DISTRIBUTION USING SIMULATED TWO-GROUP	C	80
C	DIFFUSION THEORY MODEL	C	90
C	-----	C	100
C		C	110
	COMMON /HEADR / NTITLE(20), AVEXP, NTSTEP	C	120
	COMMON /FUEL / FTYPE(N02), BU(N01), VOID(N01)	C	130
	INTEGER FTYPE	C	140
	COMMON /NODAL / DF (N01), DT (N01), KSFF (N01)	C	150
\$	KSFT (N01), TFR (N01), NSF1G (N01)	C	160
\$	C2 (N01), C4 (N01), C6 (N03)	C	170
\$	C7 (N01)	C	180
	REAL KSFF, KSFT, NSF1G	C	190
	COMMON /GEOM / NODES , NP , NDSMNP , NX	C	200
\$	NZ , NXM1 , GX , GZ	C	210
\$	NCOL (N04)	C	220
	COMMON /BC / AF1, AF3, AF5, AF6, AFQ(N04),	C	230
\$	BF1, BF3, BF5, BF6, BFQ(N04),	C	240
\$	AT1, AT3, AT5, AT6, ATQ(N04),	C	250
\$	BT1, BT3, BT5, BT6, BTQ(N04)	C	260
	EQUIVALENCE (AFQ(1), AF2), (AFQ(2), AF4),	C	270
\$	(BFQ(1), BF2), (BFQ(2), BF4),	C	280
\$	(ATQ(1), AT2), (ATQ(2), AT4),	C	290
\$	(BTQ(1), BT2), (BTQ(2), BT4)	C	300
	COMMON /CONVRG/ MAXI, MAXO, MAXP, EPSO, EPSP, ALPHA, SOR	C	310
	LOGICAL SOR	C	320
	COMMON /FLUX / POWER(N01), FASTA(N01), THERMA(N01), F(N01)	C	330
	COMMON /PROB / INTTYP, QTRKOR, NOHAL, NOCROD, NOVOID, IEDIT(6)	C	340
\$	NOXE, FIRST	C	350
	LOGICAL QTRKOR, NOHAL, NOCROD, NOVOID, NOXE, FIRST	C	360

COMMON /WEIGHT/ R, CF, CT, RCF, RCT, B1N, B2N	C	370
DIMENSION T(N01), S(N01)	C	380
REAL KEFF, LAMDA	C	390
DIMENSION FLX(N01), FO(N01), SAVE(N01), PO(N01)	C	400
EQUIVALENCE (FO(1), PO(1), T(1)), (FASTA(1), SAVE(1))	C	410
LOGICAL CONVSR, FLAG1, FLAG2	C	420
C	C	430
C-----	C	440
C	C	450
C C2, C4, C6, &C7 ARE FAST FLUX COUPLING COEFFICENTS	C	460
C F = POINTWISE FAST FLUX	C	470
C T = POINTWISE THERMAL FLUX	C	480
C FASTA = NODE AVERAGED FAST FLUX	C	490
C THERMA = NODE AVERAGED THERMAL FLUX	C	500
C POWER = RELATIVE POWER OF NODE	C	510
C	C	520
C-----	C	530
C	C	540
MAXOUT = MAXO	C	550
FLAG1 = .FALSE.	C	560
CALL INTERP	C	570
FIRST = .FALSE.	C	580
ITPOWR = 1	C	590
KEFF = 1.	C	600
SR = 0.	C	610
TLFIS = 0.	C	620
DO 603 L=1,NODES	C	630
S(L) = F(L)*NSF1G(L)	C	640
603 TLFIS = TLFIS + S(L)	C	650
IF ( .NOT. NOHAL ) GO TO 602	C	660
CALL HEADER (1)	C	670
WRITE (6,1000)	C	680
C BEGINNING OF POWER ITERATION	C	690
602 ITOUT = 0	C	700
CONVSR = .FALSE.	C	710
RSQ = 1.	C	720

IF ( NOHAL ) WRITE (6,1003) ITPOWR	C	730
C=====	C	740
C	C	750
C BEGINING OF OUTER ITERATION	C	760
500 ITIN = 0	C	770
C=====	C	780
C	C	790
C	C	800
C BEGINING OF INNER ITERATION	C	810
502 F(1) = (S(1)	+ C2(1)*F(NX+1)	C 820
\$	+ C4(1)*F(2 )	C 830
\$	+ C6(1)*F(1+NP) )/C7(1)	C 840
DO 2 L=2,NXM1	C	850
2 F(L) = (S(L)	+ C2(L)*F(L+NX)	C 860
\$	+ C4(L-1)*F(L-1 )	C 870
\$	+ C4(L)*F(L+1 )	C 880
\$	+ C6(L)*F(L+NP) )/C7(L)	C 890
F(NX) = (S(NX)	+ C2(NX)*F(NX+NX)	C 900
\$	+ C4(NXM1)*F(NXM1)	C 910
\$	+ C6(NX)*F(NX+NP) )/C7(NX)	C 920
I2 = NX	C	930
LR = NX	C	940
DO 6 I=2,NXM1	C	950
I1 = I2	C	960
I2 = NCOL(I)	C	970
LL = LR+1	C	980
LR = LR+I2	C	990
LRM1 = LR-1	C	1000
LLP1 = LL+1	C	1010
F(LL) = (S(LL) + C2(LL-I1)*F(LL-I1) + C2(LL)*F(LL+I2)	C	1020
\$	+ C4(LL)*F(LLP1)	C 1030
\$	+ C6(LL)*F(LL+NP) )/C7(LL)	C 1040
IF ( LLP1 .EQ. LR) GO TO 6	C	1050
DO 4 L=LLP1,LRM1	C	1060
4 F(L) = (S(L) + C2(L-I1)*F(L-I1) + C2(L)*F(L+I2)	C	1070
\$	+ C4(L-1)*F(L-1 )	C 1080
\$	+ C4(L)*F(L+1 )	
\$	+ C6(L)*F(L+NP) )/C7(L)	

6	F(LR) = (S(LR) + C2(LR-I1)*F(LR-I1) + C2(LR)*F(LR+I2)	C	1090
	\$ + C4(LRM1)*F(LRM1)	C	1100
	\$ + C6(LR)*F(LR+NP)/C7(LR)	C	1110
	I1 = I2	C	1120
	LL = LR+1	C	1130
	LR = NP	C	1140
	LRM1 = LR-1	C	1150
	LLP1 = LL+1	C	1160
	IF ( LRM1-LL) 10,12,14	C	1170
10	F(LL) = (S(LL) + C2(LL-I1)*F(LL-I1)	C	1180
	\$ + C6(LL)*F(LL+NP)/C7(LL)	C	1190
	GO TO 20	C	1200
12	F(LL) = (S(LL) + C2(LL-I1)*F(LL-I1)	C	1210
	\$ + C4(LL)*F(LLP1)	C	1220
	\$ + C6(LL)*F(LL+NP)/C7(LL)	C	1230
	GO TO 18	C	1240
14	F(LL) = (S(LL) + C2(LL-I1)*F(LL-I1)	C	1250
	\$ + C4(LL)*F(LLP1)	C	1260
	\$ + C6(LL)*F(LL+NP)/C7(LL)	C	1270
	DO 16 L=LLP1,LRM1	C	1280
16	F(L) = (S(L) + C2(L-I1)*F(L-I1)	C	1290
	\$ + C4(L-1)*F(L-1) + C4(L)*F(L+1)	C	1300
	\$ + C6(L)*F(L+NP)/C7(L)	C	1310
18	F(LR) = (S(LR) + C2(LR-I1)*F(LR-I1)	C	1320
	\$ + C4(LRM1)*F(LRM1)	C	1330
	\$ + C6(LR)*F(LR+NP)/C7(LR)	C	1340
20	DO 40 K=3,NZ	C	1350
	I2 = NX	C	1360
	LL = LR+1	C	1370
	LR = LR+I2	C	1380
	LRM1 = LR-1	C	1390
	LLP1 = LL+1	C	1400
	F(LL) = (S(LL) + C2(LL)*F(LL+I2)	C	1410
	\$ + C4(LL)*F(LLP1)	C	1420
	\$ + C6(LL-NP)*F(LL-NP) + C6(LL)*F(LL+NP)/C7(LL)	C	1430
	DO 22 L=LLP1,LRM1	C	1440

22	F(L) = (S(L)	+ C2(L)*F(L+I2)	C	1450
	\$	+ C4(L-1)*F(L-1) + C4(L)*F(L+1)	C	1460
	\$	+ C6(L-NP)*F(L-NP) + C6(L)*F(L+NP) )/C7(L)	C	1470
	F(LR) = (S(LR)	+ C2(LR)*F(LR+I2)	C	1480
	\$	+ C4(LRM1)*F(LRM1)	C	1490
	\$	+ C6(LR-NP)*F(LR-NP) + C6(LR)*F(LR+NP) )/C7(LR)	C	1500
	DO 26 I=2,NXM1		C	1510
	I1 = I2		C	1520
	I2 = NCOL(I)		C	1530
	LL = LR+1		C	1540
	LR = LR+I2		C	1550
	LRM1 = LR-1		C	1560
	LLP1 = LL+1		C	1570
	F(LL) = (S(LL) + C2(LL-I1)*F(LL-I1) + C2(LL)*F(LL+I2)		C	1580
	\$	+ C4(LL)*F(LLP1)	C	1590
	\$	+ C6(LL-NP)*F(LL-NP) + C6(LL)*F(LL+NP) )/C7(LL)	C	1600
	IF ( LLP1 .EQ. LR) GO TO 26		C	1610
	DO 24 L=LLP1,LRM1		C	1620
24	F(L) = (S(L) + C2(L-I1)*F(L-I1) + C2(L)*F(L+I2)		C	1630
	\$	+ C4(L-1)*F(L-1) + C4(L)*F(L+1)	C	1640
	\$	+ C6(L-NP)*F(L-NP) + C6(L)*F(L+NP) )/C7(L)	C	1650
26	F(LR) = (S(LR) + C2(LR-I1)*F(LR-I1) + C2(LR)*F(LR+I2)		C	1660
	\$	+ C4(LRM1)*F(LRM1)	C	1670
	\$	+ C6(LR-NP)*F(LR-NP) + C6(LR)*F(LR+NP) )/C7(LR)	C	1680
	I1 = I2		C	1690
	LL = LR+1		C	1700
	LR = LR+NCOL(NX)		C	1710
	LRM1 = LR-1		C	1720
	LLP1 = LL+1		C	1730
	IF ( LRM1-LL) 30,32,34		C	1740
30	F(LL) = (S(LL) + C2(LL-I1)*F(LL-I1)		C	1750
	\$	+ C6(LL-NP)*F(LL-NP) + C6(LL)*F(LL+NP) )/C7(LL)	C	1760
	GO TO 40		C	1770
32	F(LL) = (S(LL) + C2(LL-I1)*F(LL-I1)		C	1780
	\$	+ C4(LL)*F(LLP1)	C	1790
	\$	+ C6(LL-NP)*F(LL-NP) + C6(LL)*F(LL+NP) )/C7(LL)	C	1800

	GO TO 38	C	1810
34	F(LL) = (S(LL) + C2(LL-I1)*F(LL-I1)	C	1820
	\$ + C4(LL)*F(LLP1)	C	1830
	\$ + C6(LL-NP)*F(LL-NP) + C6(LL)*F(LL+NP) )/C7(LL)	C	1840
	DO 36 L=LLP1,LRM1	C	1850
36	F(L) = (S(L) + C2(L-I1)*F(L-I1)	C	1860
	\$ + C4(L-1)*F(L-1) + C4(L)*F(L+1)	C	1870
	\$ + C6(L-NP)*F(L-NP) + C6(L)*F(L+NP) )/C7(L)	C	1880
38	F(LR) = (S(LR) + C2(LR-I1)*F(LR-I1)	C	1890
	\$ + C4(LRM1)*F(LRM1)	C	1900
	\$ + C6(LR-NP)*F(LR-NP) + C6(LR)*F(LR+NP) )/C7(LR)	C	1910
40	CONTINUE	C	1920
	LL = LR+1	C	1930
	LR = LR+NX	C	1940
	LLP1 = LL+1	C	1950
	LRM1 = LR-1	C	1960
	F(LL) = (S(LL) + C2(LL)*F(LL+NX)	C	1970
	\$ + C4(LL)*F(LLP1)	C	1980
	\$ + C6(LL-NP)*F(LL-NP) )/C7(LL)	C	1990
	DO 42 L=LLP1,LRM1	C	2000
42	F(L) = (S(L) + C2(L)*F(L+NX)	C	2010
	\$ + C4(L-1)*F(L-1) + C4(L)*F(L+1)	C	2020
	\$ + C6(L-NP)*F(L-NP) )/C7(L)	C	2030
	F(LR) = (S(LR) + C2(LR)*F(LR+NX)	C	2040
	\$ + C4(LRM1)*F(LRM1)	C	2050
	\$ + C6(LR-NP)*F(LR-NP) )/C7(LR)	C	2060
	I2 = NX	C	2070
	DO 54 I=2,NXM1	C	2080
	I1 = I2	C	2090
	I2 = NCOL(I)	C	2100
	LL = LR+1	C	2110
	LR = LR+I2	C	2120
	LLP1 = LL+1	C	2130
	LRM1 = LR-1	C	2140
	F(LL) = (S(LL) + C2(LL-I1)*F(LL-I1) + C2(LL)*F(LL+I2)	C	2150
	\$ + C4(LL)*F(LLP1)	C	2160

	\$	+ C6(LL-NP)*F(LL-NP)	) /C7(LL)	C	2170
		IF ( LRM1 .EQ. LL) GO TO 48		C	2180
		DO 46 L=LLP1,LRM1		C	2190
		IF ( L+I2-NODES) 942,942,44		C	2200
942		F(L) = (S(L) + C2(L-I1)*F(L-I1) + C2(L)*F(L+I2)		C	2210
	\$	+ C4(L-1 )*F(L-1 ) + C4(L)*F(L+1)		C	2220
	\$	+ C6(L-NP)*F(L-NP)	) /C7(L)	C	2230
		GO TO 46		C	2240
44		F(L) = (S(L) + C2(L-I1)*F(L-I1)		C	2250
	\$	+ C4(L-1 )*F(L-1 ) + C4(L)*F(L+1)		C	2260
	\$	+ C6(L-NP)*F(L-NP)	) /C7(L)	C	2270
46		CONTINUE		C	2280
48		IF ( LR+I2-NODES) 50,50,52		C	2290
50		F(LR) = (S(LR) + C2(LR-I1)*F(LR-I1) + C2(LR)*F(LR+I2)		C	2300
	\$	+ C4(LRM1 )*F(LRM1 )		C	2310
	\$	+ C6(LR-NP)*F(LR-NP)	) /C7(LR)	C	2320
		GO TO 54		C	2330
52		F(LR) = (S(LR) + C2(LR-I1)*F(LR-I1)		C	2340
	\$	+ C4(LRM1 )*F(LRM1 )		C	2350
	\$	+ C6(LR-NP)*F(LR-NP)	) /C7(LR)	C	2360
54		CONTINUE		C	2370
		I1 = I2		C	2380
		IF ( 2-NCOL(NX) ) 60,58,56		C	2390
56		F(NODES) = (S(NODES) + C2(NODES-I1)*F(NODES-I1)		C	2400
	\$	+ C6(NDSMNP)*F(NDSMNP)	) /C7(NODES)	C	2410
		GO TO 138		C	2420
58		LL = NODES-1		C	2430
		F(LL) = (S(LL) + C2(LL-I1)*F(LL-I1)		C	2440
	\$	+ C4(LL)*F(NODES)		C	2450
	\$	+ C6(LL-NP)*F(LL-NP)	) /C7(LL)	C	2460
		GO TO 64		C	2470
60		LL = LR+1		C	2480
		LLP1 = LL+1		C	2490
		LRM1 = NODES-1		C	2500
		F(LL) = (S(LL) + C2(LL-I1)*F(LL-I1)		C	2510
	\$	+ C4(LL)*F(LLP1)		C	2520

	\$	+ C6(LL-NP)*F(LL-NP)	) / C7(LL)	C	2530
		DO 62 L=LLP1,LRM1		C	2540
62		F(L) = (S(L) + C2(L-11)*F(L-11)		C	2550
	\$	+ C4(L-1 )*F(L-1 ) + C4(L)*F(L+1)		C	2560
	\$	+ C6(L-NP)*F(L-NP)	) / C7(L)	C	2570
64		F(NODES) = (S(NODES) + C2(NODES-11)*F(NODES-11)		C	2580
	\$	+ C4(NODES-1 )*F(NODES-1 )		C	2590
	\$	+ C6(NDSMNP)*F(NDSMNP)	) / C7(NODES)	C	2600
	C	=====		C	2610
	C			C	2620
	C			C	2630
138		ITIN = ITIN + 1		C	2640
	C	-----		C	2650
	C			C	2660
	C	SUCCESSIVE OVER-RELAXATION ROUTINE		C	2670
		IF ( .NOT. SOR ) GO TO 1396		C	2680
	C			C	2690
		IF ( CONVSR ) GO TO 804		C	2700
	C			C	2710
	C	CALCULATE SPECTRAL RADIUS		C	2720
		RSQ0 = RSQ		C	2730
		RSQ = 0.		C	2740
		DO 800 L=1,NODES		C	2750
		RSQ = RSQ + ( F(L) - FO(L) )**2		C	2760
800		FO(L) = F(L)		C	2770
		SR0 = SR		C	2780
		SR = SQRT (RSQ/RSQ0)		C	2790
	C			C	2800
		IF ( ABS (1.-SR0/SR) .LE. .001 ) GO TO 803		C	2810
		IF ( ITIN .GE. 20 ) GO TO 803		C	2820
		GO TO 502		C	2830
	C			C	2840
803		CONVSR = .TRUE.		C	2850
		IF ( SR .GE. 1. ) SOR = .FALSE.		C	2860
		IF ( .NOT. SOR ) GO TO 1396		C	2870
		WRITE (6,1005) SR		C	2880



IF ( SR .LT. 1. ) GO TO 806	C	2890
ALP = 2./(1. + SQRT (1. - SR))	C	2900
GO TO 1396	C	2910
806 ALP = 1.	C	2920
GO TO 1396	C	2930
804 DO 805 L=1,NODES	C	2940
F(L) = FO(L) + ALP* (F(L) - FO(L))	C	2950
805 FO(L) = F(L)	C	2960
C	C	2970
1396 IF ( ITIN.LT.MAXI) GO TO 502	C	2980
C-----	C	2990
C	C	3000
CALCULATE K-EFF	C	3010
TLFISO = TLFIS	C	3020
TLFIS = 0.	C	3030
DO 550 L=1,NODES	C	3040
550 TLFIS = TLFIS+F(L)*NSFIG(L)	C	3050
LAMDA = TLFIS/TLFISO	C	3060
KEFF = KEFF*LAMDA	C	3070
TESTO = ABS (1. - TLFISO/TLFIS)	C	3080
ITOUT = ITOUT+1	C	3090
IF ( ITOUT .EQ. 1) GO TO 554	C	3100
C=====	C	3110
C	C	3120
C	C	3130
C ACCELERATE SOURCE BY SUCCESSIVE OVER-RELAXATION	C	3140
TFISOR = 0.	C	3150
DO 552 L=1,NODES	C	3160
SAVE(L) = SAVE(L)+ALPHA*(F(L)-SAVE(L))	C	3170
552 TFISOR = TFISOR+SAVE(L)*NSFIG(L)	C	3180
GO TO 558	C	3190
554 TFISOR = TLFIS	C	3200
DO 556 L=1,NODES	C	3210
556 SAVE(L) = F(L)	C	3220
558 TEMP = TLFIS/TFISOR/KEFF	C	3230
DO 560 L=1,NODES	C	3240

S(L) = SAVE(L)*NSFIG(L)*TEMP	C	3250
560 SAVE(L) = F(L)	C	3260
TLFISO = TLFIS	C	3270
C=====	C	3280
C	C	3290
CONVERGENCE CHECK	C	3300
WRITE (6,1002) ITOUT,ITIN,KEFF,TESTO	C	3310
IF ( ITOUT .GE. MAXOUT) GO TO 200	C	3320
IF ( TESTO .GT. EPS0) GO TO 500	C	3330
C=====	C	3340
C	C	3350
C NODAL WEIGHTING SECTION	C	3360
C-----	C	3370
C	C	3380
CALCULATE AVERAGED FAST FLUX	C	3390
200 DO 202 L=1,NODES	C	3400
FASTA(L) = BIN*F(L)	C	3410
202 FLX(L) = CF*F(L)*DF(L)	C	3420
C FACE 5 BOUNDARY	C	3430
DO 204 L=1,NP	C	3440
204 FASTA(L) = FASTA(L)+RCF*(AF5+BF5*VOID(L))*F(L)	C	3450
C FACE 6 BOUNDARY	C	3460
L1 = NDSMNP+1	C	3470
DO 206 L=L1,NODES	C	3480
206 FASTA(L) = FASTA(L)+RCF*(AF6+BF6*VOID(L))*F(L)	C	3490
C HORIZONTAL PLANES	C	3500
DO 208 L=1,NDSMNP	C	3510
LP = L+NP	C	3520
FF = R*(FLX(L)+FLX(LP))/(DF(L)+DF(LP))	C	3530
FASTA(L) = FASTA(L)+FF	C	3540
208 FASTA(LP) = FASTA(LP)+FF	C	3550
C FACE 1 BOUNDARY	C	3560
DO 210 LB=1,NX	C	3570
LT = LB+NDSMNP	C	3580
DO 210 L=LB,LT,NP	C	3590
210 FASTA(L) = FASTA(L)+CF*(AF1+BF1*VOID(L))*F(L)	C	3600

C	IF ( QTRKOR ) GO TO 218	C	3610
C	RECTANGULAR PLANE GEOMETRY	C	3620
C	FACE 2	C	3630
	L1 = NP-NXM1	C	3640
	DO 214 LB=L1,NP	C	3650
	LT = LB+NDSMNP	C	3660
	DO 214 L=LB,LT,NP	C	3670
214	FASTA(L) = FASTA(L)+CF*(AF2+BF2*VOID(L))*F(L)	C	3680
C	FACE 4	C	3690
	DO 216 LB=NX,NP,NX	C	3700
	LT = LB+NDSMNP	C	3710
	DO 216 L=LB,LT,NP	C	3720
216	FASTA(L) = FASTA(L)+CF*(AF4+BF4*VOID(L))*F(L)	C	3730
	GO TO 272	C	3740
		C	3750
C		C	3760
C	QUARTER CORE PLANE GEOMETRY	C	3770
C	FACE 2	C	3780
218	J = 0	C	3790
	L1 = NP-NCOL(NX)+1	C	3800
	L2 = NP	C	3810
	DO 220 LB=L1,L2	C	3820
	LT = LB+NDSMNP	C	3830
	J = J+1	C	3840
	AF = AFQ(J)	C	3850
	BF = BFQ(J)	C	3860
	DO 220 L=LB,LT,NP	C	3870
220	FASTA(L) = FASTA(L)+CF*(AF+BF*VOID(L))*F(L)	C	3880
	I = NX	C	3890
	NCL = NCOL(NX)	C	3900
	DO 224 IC=2,NX	C	3910
	I = I-1	C	3920
	NCI = NCOL(I)	C	3930
	L2 = L2-NCL	C	3940
	L1 = L2-NCI+NCL+1	C	3950
	IF ( NCI .EQ. NCL ) GO TO 224	C	3960

DO 222 LB=L1,L2	C	3970
LT = LB+NDSMNP	C	3980
J = J+1	C	3990
AF = AFQ(J)	C	4000
BF = BFQ(J)	C	4010
DO 222 L=LB,LT,NP	C	4020
222 FASTA(L) = FASTA(L)+CF*(AF+BF*VOID(L))*F(L)	C	4030
224 NCL = NCI	C	4040
C FACE 4	C	4050
LB = 0	C	4060
DO 226 I=1,NX	C	4070
LB = LB+NCOL(I)	C	4080
LT = LB+NDSMNP	C	4090
AF = AFQ(I)	C	4100
BF = BFQ(I)	C	4110
DO 226 L=LB,LT,NP	C	4120
226 FASTA(L) = FASTA(L)+CF*(AF+BF*VOID(L))*F(L)	C	4130
C	C	4140
C FACE 3 BOUNDARY	C	4150
272 LB = 1	C	4160
DO 276 I=1,NX	C	4170
LT = LB+NDSMNP	C	4180
DO 274 L=LB,LT,NP	C	4190
274 FASTA(L) = FASTA(L)+CF*(AF3+BF3*VOID(L))*F(L)	C	4200
276 LB = LB+NCOL(I)	C	4210
C J FACES	C	4220
L2 = -1	C	4230
DO 280 I=1,NX	C	4240
L1 = L2+2	C	4250
L2 = L2+NCOL(I)	C	4260
IF ( L2 .LT. L1) GO TO 280	C	4270
DO 278 LB=L1,L2	C	4280
LT = LB+NDSMNP	C	4290
DO 278 L=LB,LT,NP	C	4300
LP = L+1	C	4310
FF = (FLX(L)+FLX(LP))/(DF(L)+DF(LP))	C	4320

FASTA(L) = FASTA(L)+FF	C	4330
278 FASTA(LP) = FASTA(LP)+FF	C	4340
280 CONTINUE	C	4350
C I FACES	C	4360
LR = 0	C	4370
NCI = NX	C	4380
DO 284 I=1,NXM1	C	4390
NCIP = NCOL(I+1)	C	4400
L2 = LR+NCIP	C	4410
L1 = LR+1	C	4420
LR = LR+NCI	C	4430
DO 282 LB=L1,L2	C	4440
LT = LB+NDSMNP	C	4450
DO 282 L=LB,LT,NP	C	4460
LP = L+NCI	C	4470
FF = (FLX(L)+FLX(LP))/(DF(L)+DF(LP))	C	4480
FASTA(L) = FASTA(L)+FF	C	4490
282 FASTA(LP) = FASTA(LP)+FF	C	4500
284 NCI = NCIP	C	4510
C=====	C	4520
C	C	4530
CALCULATE AVERAGED THERMAL FLUX	C	4540
DO 302 L=1,NODES	C	4550
T(L) = F(L)*TFR(L)	C	4560
THERMA(L) = B2N*T(L)	C	4570
302 FLX(L) = CT*T(L)*DT(L)	C	4580
C FACE 5 BOUNDARY	C	4590
DO 304 L=1,NP	C	4600
304 THERMA(L) = THERMA(L)+RCT*(AT5+BT5*VOID(L))*T(L)	C	4610
C FACE 6 BOUNDARY	C	4620
L1 = NDSMNP+1	C	4630
DO 306 L=L1,NODES	C	4640
306 THERMA(L) = THERMA(L)+RCT*(AT6+BT6*VOID(L))*T(L)	C	4650
C HORIZONTAL PLANES	C	4660
DO 308 L=1,NDSMNP	C	4670
LP = L+NP	C	4680

FF = R*(FLX(L)+FLX(LP))/(DT(L)+DT(LP))	C	4690
THERMA(L) = THERMA(L)+FF	C	4700
308 THERMA(LP) = THERMA(LP)+FF	C	4710
C FACE 1 BOUNDARY	C	4720
DO 310 LB=1,NX	C	4730
LT = LB+NDSMNP	C	4740
DO 310 L=LB,LT,NP	C	4750
310 THERMA(L) = THERMA(L)+CT*(AT1+BT1*VOID(L))*T(L)	C	4760
C	C	4770
IF ( QTRKOR ) GO TO 318	C	4780
C RECTANGULAR PLANE GEOMETRY	C	4790
C FACE 2 BOUNDARY	C	4800
L1 = NP-NXM1	C	4810
DO 314 LB=L1,NP	C	4820
LT = LB+NDSMNP	C	4830
DO 314 L=LB,LT,NP	C	4840
314 THERMA(L) = THERMA(L)+CT*(AT2+BT2*VOID(L))*T(L)	C	4850
C FACE 4 BOUNDARY	C	4860
DO 316 LB=NX,NP,NX	C	4870
LT = LB+NDSMNP	C	4880
DO 316 L=LB,LT,NP	C	4890
316 THERMA(L) = THERMA(L)+CT*(AT4+BT4*VOID(L))*T(L)	C	4900
GO TO 372	C	4910
C	C	4920
C QUARTER CORE PLANE GEOMETRY	C	4930
C FACE 2 BOUNDARY	C	4940
318 J = 0	C	4950
L1 = NP-NCOL(NX)+1	C	4960
L2 = NP	C	4970
DO 320 LB=L1,L2	C	4980
LT = LB+NDSMNP	C	4990
J = J+1	C	5000
AT = ATQ(J)	C	5010
BT = BTQ(J)	C	5020
DO 320 L=LB,LT,NP	C	5030
320 THERMA(L) = THERMA(L)+CT*(AT+BT*VOID(L))*T(L)	C	5040

I = NX	C 5050
NCL = NCOL(NX)	C 5060
DO 324 IC=2,NX	C 5070
I = I-1	C 5080
NCI = NCOL(I)	C 5090
L2 = L2-NCL	C 5100
L1 = L2-NCI+NCL+1	C 5110
IF ( NCI .EQ. NCL) GO TO 324	C 5120
DO 322 LB=L1,L2	C 5130
LT = LB+NDSMNP	C 5140
J = J+1	C 5150
AT = ATQ(J)	C 5160
BT = BTQ(J)	C 5170
DO 322 L=LB,LT,NP	C 5180
322 THERMA(L) = THERMA(L)+CT*(AT+BT*VOID(L))*T(L)	C 5190
324 NCL = NCI	C 5200
C FACE 4 BOUNDARY	C 5210
LB = 0	C 5220
DO 326 I=1,NX	C 5230
LB = LB+NCOL(I)	C 5240
LT = LB+NDSMNP	C 5250
AT = ATQ(I)	C 5260
BT = BTQ(I)	C 5270
DO 326 L=LB,LT,NP	C 5280
326 THERMA(L) = THERMA(L)+CT*(AT+BT*VOID(L))*T(L)	C 5290
C	C 5300
C FACE 3 BOUNDARY	C 5310
372 LB = 1	C 5320
DO 376 I=1,NX	C 5330
LT = LB+NDSMNP	C 5340
DO 374 L=LB,LT,NP	C 5350
374 THERMA(L) = THERMA(L)+CT*(AT3+BT3*VOID(L))*T(L)	C 5360
376 LB = LB+NCOL(I)	C 5370
C J FACES	C 5380
L2 = -1	C 5390
DO 380 I=1,NX	C 5400

L1 = L2+2	C	5410
L2 = L2+NCOL(I)	C	5420
IF ( L2 .LT. L1) GO TO 380	C	5430
DO 378 LB=L1,L2	C	5440
LT = LB+NDSMNP	C	5450
DO 378 L=LB,LT,NP	C	5460
LP = L+1	C	5470
FF = (FLX(L)+FLX(LP))/(DT(L)+DT(LP))	C	5480
THERMA(L) = THERMA(L)+FF	C	5490
378 THERMA(LP) = THERMA(LP)+FF	C	5500
380 CONTINUE	C	5510
C I FACES	C	5520
LR = 0	C	5530
NCI = NX	C	5540
DO 384 I=1,NXM1	C	5550
NCIP = NCOL(I+1)	C	5560
L2 = LR+NCIP	C	5570
L1 = LR+1	C	5580
LR = LR+NCI	C	5590
DO 382 LB=L1,L2	C	5600
LT = LB+NDSMNP	C	5610
DO 382 L=LB,LT,NP	C	5620
LP = L+NCI	C	5630
FF = (FLX(L)+FLX(LP))/(DT(L)+DT(LP))	C	5640
THERMA(L) = THERMA(L)+FF	C	5650
382 THERMA(LP) = THERMA(LP)+FF	C	5660
384 NCI = NCIP	C	5670
C=====	C	5680
C	C	5690
CALCULATE RELATIVE POWER	C	5700
DO 400 L=1,NODES	C	5710
PO(L) = POWER(L)	C	5720
400 POWER(L) = FASTA(L)*KSFF(L) + THERMA(L)*KSFT(L)	C	5730
CALL NORM (POWER, NODES)	C	5740
IF ( .NOT. NOHAL ) RETURN	C	5750
CHECK CONVERGENCE	C	5760



FLAG2 = FLAG1	C	5770
TESTP0 = TESTP	C	5780
TESTP = 0.	C	5790
DO 402 L=1,NODES	C	5800
TEST = ABS (1.-PO(L)/POWER(L))	C	5810
IF ( TEST .GT. TESTP) TESTP = TEST	C	5820
402 CONTINUE	C	5830
IF ( ITPWR .EQ. 1 ) GO TO 403	C	5840
C IF VOID ITERATION IS DIVERGING DECREASE NUMBER OF OUTERS PER VOID IT.	C	5850
FLAG1 = TESTP .GT. TESTP0	C	5860
IF ( FLAG1 .AND. FLAG2 ) MAXOUT = MAXOUT - 1	C	5870
WRITE (6,1006) TESTP	C	5880
IF ( TESTP .LE. EPSP) RETURN	C	5890
IF ( ITPWR .GE. MAXP) GO TO 412	C	5900
403 ITPWR = ITPWR+1	C	5910
C	C	5920
IF ( NOVOID ) GO TO 404	C	5930
CALL VOIDFR	C	5940
404 IF ( NOXE ) GO TO 602	C	5950
405 CALL INTERP	C	5960
GO TO 602	C	5970
412 WRITE (6,1004) ITPWR	C	5980
RETURN	C	5990
C	C	6000
C-----	C	6010
C	C	6020
1000 FORMAT (10X,'====='/10X,'ITERATION MONITOR'/10X,'=====	C	6030
\$===== '/1X,T11,'VOID',T22,'OUTER',T33,'INNER',	C	6040
\$ /1X,T11,3('ITERATION '),T47,'K-EFF',T59,'TEST0',T71,'TES	C	6050
STP'/10X,3('----- '), '----- ',1X,2('----- '))//)	C	6060
1002 FORMAT (1X,T20,2I11,F10.4,T52,E12.3)	C	6070
1003 FORMAT (1X,T10,I9)	C	6080
1004 FORMAT (1X,'WARNING: POWER ITERATION HAS NOT CONVERGED IN ',I5,	C	6090
\$' ITERATIONS. FURTHER ITERATION HAS BEEN SUPPRESSED.')	C	6100
1005 FORMAT (1H+,T81,'SPECTRAL RADIUS =',F10.3)	C	6110
1006 FORMAT (1H+,T64,E12.3)	C	6120

C

END

C 6130

C 6140

C*	=====	D	10
C	=====	D	20
C		D	30
C		D	40
C	SUBROUTINE HEADER (N)	D	50
C		D	60
C	-----	D	70
C	ROUTINE TO PRINT PAGE HEADING	D	80
C	-----	D	90
C		D	100
C	COMMON /HEADR / NTITLE(20), AVEXP, NTSTEP	D	110
C	DATA NPAGE/1/	D	120
C		D	130
C	-----	D	140
C		D	150
C	GO TO (10,20), N	D	160
10	WRITE (6,2) NTSTEP, NTITLE, NPAGE	D	170
	GO TO 30	D	180
20	WRITE (6,4) NTSTEP, NTITLE, NPAGE, AVEXP	D	190
30	NPAGE = NPAGE+1	D	200
	RETURN	D	210
C		D	220
C	-----	D	230
C		D	240
2	FORMAT (1H1,'TIME STEP #',I4,9X,'<< ',20A4,' >>',11X,'PAGE',I5//	D	250
	\$/)	D	260
4	FORMAT (1H1,'TIME STEP #',I4,9X,'<< ',20A4,' >>',11X,'PAGE',I5/	D	270
	\$1X,'EXPOSURE =',F7.0,' MWD/MT'///)	D	280
C		D	290
	END	D	300

C#=====	E	10
C=====	E	20
C	E	30
C	E	40
SUBROUTINE ERROR (N)	E	50
C	E	60
C-----	E	70
C ROUTINE TO PRINT ERROR MESSAGE AND ABORT JOB	E	80
C-----	E	90
C	E	100
GO TO (11,12,13,14,15,16,17,18,19,20), N	E	110
11      WRITE (6,1)	E	120
STOP	E	130
12      WRITE (6,2)	E	140
STOP	E	150
13      WRITE (6,3)	E	160
STOP	E	170
14      WRITE (6,4)	E	180
STOP	E	190
15      WRITE (6,5)	E	200
STOP	E	210
16      WRITE (6,6)	E	220
STOP	E	230
17      WRITE (6,7)	E	240
STOP	E	250
18      WRITE (6,8)	E	260
STOP	E	270
19      WRITE (6,9)	E	280
STOP	E	290
20      WRITE (6,10)	E	300
STOP	E	310
C	E	320
C-----	E	330
C	E	340
1      FORMAT (//1X,'***  FORMAT ERROR IN INPUT.  JOB ABORTED.  ***')	E	350
2      FORMAT (//1X,'***  ERROR IN INPUT:  NCOL(1) .NE. NX .  JOB ABORTED	E	360

	\$.	****)				E	370
3	FORMAT (//1X,****	ERROR IN INPUT:	SUM( NCOL(I) ) .NE. NP .	JOB		E	380
	\$ABORTED.	****)				E	390
4	FORMAT (//1X,****	ERROR IN INPUT:	NZ*NP < OR > NODES.	JOB ABORT		E	400
	\$ED.	****)				E	410
5	FORMAT (//1X,****	ERROR IN INPUT:	IPTYPE=2 & IEXPTS=1.	JOB ABOR		E	420
	\$TED.	****)				E	430
6	FORMAT (//1X,****	ERROR IN INPUT:	ITSETS > DIMTS.	JOB ABORTED.		E	440
	\$ ****)					E	450
7	FORMAT (//1X,****	ERROR IN INPUT:	IEXPTS > DIMEX.	JOB ABORTED.		E	460
	\$ ****)					E	470
8	FORMAT (//1X,****	ERROR IN INPUT:	IVDPTS > DIMVD.	JOB ABORTED.		E	480
	\$ ****)					E	490
9	FORMAT (//1X,****	ERROR IN INPUT:	EXPOSURE MASK NOT IN ASCENDING			E	500
	\$ ORDER.	JOB ABORTED.	****)			E	510
10	FORMAT (//1X,****	ERROR IN INPUT:	VOID MASK NOT IN ASCENDING ORD			E	520
	\$ER.	JOB ABORTED.	****)			E	530
C						E	540
	END					E	550

C#=====	F	10
C=====	F	20
C	F	30
C	F	40
FUNCTION LOC(I,J,K)	F	50
C	F	60
C-----	F	70
C ROUTINE TO CONVERT (I,J,K) POSITION INDICES TO AN 'L' POSITION INDEX	F	80
C-----	F	90
C	F	100
COMMON /GEOM / NODES , NP , NDSMNP , NX	F	110
\$, NZ , NXM1 , GX , GZ	F	120
\$, NCOL (N04)	F	130
C	F	140
C-----	F	150
C	F	160
LOC = (K-1)*NP+J	F	170
IF ( I .EQ. 1) RETURN	F	180
IM1 = I-1	F	190
DO 10 I1=1,IM1	F	200
10 LOC = LOC+NCOL(I1)	F	210
RETURN	F	220
END	F	230

C*****	G	10
C*****	G	20
C	G	30
C	G	40
SUBROUTINE HALING (BURN)	G	50
C	G	60
C-----	G	70
C ROUTINE TO CALCULATE HALING SOLUTION FOR OPTIMUM POWER DISTRIBUTION	G	80
C-----	G	90
C	G	100
COMMON /FUEL / FTYPE(N02), BU(N01), VOID(N01)	G	110
INTEGER FTYPE	G	120
COMMON /GEOM / NODES , NP , NDSMNP , NX	G	130
\$, NZ , NXM1 , GX , GZ	G	140
\$, NCOL (N04)	G	150
COMMON /FLUX / POWER(N01), FASTA(N01), THERMA(N01), F(N01)	G	160
COMMON /CONVRG/ MAXI, MAX0, MAXP, EPS0, EPSP, ALPHA, SOR	G	170
LOGICAL SOR	G	180
DIMENSION PO(N01), EXS(N01)	G	190
C	G	200
C-----	G	210
NOCONV = 0	G	220
ITHAL = 1	G	230
TESTH = 1.E6	G	240
WRITE (6,1000)	G	250
C	G	260
DO 10 L=1,NODES	G	270
EXS(L) = BU(L)	G	280
BU(L) = EXS(L) + POWER(L)*BURN	G	290
10 PO(L) = POWER(L)	G	300
GO TO 20	G	310
C	G	320
15 CALL VOIDFR	G	330
20 WRITE (6,1003) ITHAL	G	340
CALL STG	G	350
TESTH0 = TESTH	G	360

TESTH = 0.	G	370
DO 30 L=1,NODES	G	380
POWER(L) = .9*PO(L) + .1*POWER(L)	G	390
TEST = ABS (1.-PO(L)/POWER(L))	G	400
IF ( TEST .GT. TESTH ) TESTH = TEST	G	410
PO(L) = POWER(L)	G	420
30 BU(L) = EXS(L) + POWER(L)*BURN	G	430
C	G	440
ITHAL = ITHAL + 1	G	450
WRITE (6,1006) TESTH	G	460
IF ( TESTH .LT. TESTH0 ) GO TO 50	G	470
NOCONV = NOCONV + 1	G	480
IF ( NOCONV .GE. 3 ) GO TO 60	G	490
GO TO 51	G	500
50 NOCONV = 0	G	510
51 CONTINUE	G	520
IF ( TESTH .GT. EPSP ) GO TO 15	G	530
RETURN	G	540
60 WRITE (6,110)	G	550
RETURN	G	560
C-----	G	570
C	G	580
1000 FORMAT (10X,'===== '/10X,'ITERATION MONITOR'/10X,'=====	G	590
\$===== '//1X,T11,'HALING',T22,'OUTER',T33,'INNER',	G	600
\$ /1X,T11,3('ITERATION '),T47,'K-EFF',T59,'TESTO',T71,'TES	G	610
\$TH'/10X,3('----- '), '----- ',1X,2('----- '))//)	G	620
1003 FORMAT (1X,T10,I9)	G	630
1006 FORMAT (1H+,T64,E12.3)	G	640
110 FORMAT (1X,'HALING ITERATION IS DIVERGING. FURTHER ITERATION HAS	G	650
\$BEEN SUPRESSED.')	G	660
END	G	670



C*****	H	10
C*****	H	20
C	H	30
C	H	40
SUBROUTINE NORM (A, N)	H	50
C-----	H	60
C ROUTINE TO NORMALIZE AN ARRAY TO A MEAN OF 1.00	H	70
C-----	H	80
C	H	90
DIMENSION A(N)	H	100
C	H	110
C-----	H	120
C	H	130
SUM = 0.	H	140
DO 10 L=1,N	H	150
10      SUM = SUM + A(L)	H	160
AVG = SUM/FLOAT (N)	H	170
DO 12 L=1,N	H	180
12      A(L) = A(L)/AVG	H	190
RETURN	H	200
END	H	210

C*****	I	10
C*****	I	20
C	I	30
C	I	40
SUBROUTINE VOIDFR	I	50
C	I	60
C-----	I	70
C ROUTINE TO CALCULATE CHANNEL FLOW AND MODERATOR VOID DISTRIBUTION	I	80
C-----	I	90
C	I	100
COMMON /FUEL / FTYPE(N02), BU(N01), VOID(N01)	I	110
INTEGER FTYPE	I	120
COMMON /GEOM / NODES , NP , NDSMNP , NX	I	130
\$, NZ , NXM1 , GX , GZ	I	140
\$, NCOL (N04)	I	150
COMMON /FLUX / POWER(N01), FASTA(N01), THERMA(N01), F(N01)	I	160
COMMON /PROB / INTTYP, QTRKOR, NOHAL, NOCROD, NOVOID, IEDIT(6)	I	170
\$, NOXE, FIRST	I	180
LOGICAL QTRKOR, NOHAL, NOCROD, NOVOID, NOXE, FIRST	I	190
COMMON /THERMO/ HFG, XIN, AC1, AC2, AC3, AREA, AVFLUX, AVPWR, PSIA	I	200
\$, B1(4), B2(4), B3(4), CTYPE(N02), FLOW(N02), TFLOW	I	210
\$, PCTPWR	I	220
INTEGER CTYPE	I	230
C	I	240
C-----	I	250
C	I	260
C FLOW = COOLANT FLOW IN CHANNEL (LBM/HR)	I	270
C CTYPE = FLOW CHANNEL TYPE ARRAY	I	280
C PCTPWR = PER CENT OF RATED POWER	I	290
C TFLOW = TOTAL INCORE COOLANT FLOW (LBM/HR)	I	300
C PAV = RELATIVE CHANNEL POWER	I	310
C B1,B2,B3 = COEFFICIENTS USED TO CALCULATE RELATIVE CHANNEL FLOW	I	320
C AS A FUNCTION OF AVERAGE CHANNEL POWER:	I	330
C FLOW = B1 + B2*PAV + B3*PAV**2	I	340
C AVPWR = AVERAGE POWER PER NODE (BTU/HR)	I	350
C X = COOLANT QUALITY	I	360

C	XIN	= INLET QUALITY	I	370	
C	HFG	= HEAT OF VAPORIZATION (BTU/LBM)	I	380	
C	AC1,AC2	= ARMAND CORRELATION COEFFICIENTS	I	390	
C	AC3	= VF/VG	I	400	
C			I	410	
C	-----			I	420
C			I	430	
	AC4	= 1.-AC3	I	440	
	CALCULATE CHANNEL FLOW		I	450	
C			I	460	
	DO 10 LB=1,NP		I	470	
	LT = LB+NDSMNP		I	480	
	CALCULATE AVERAGE CHANNEL POWER		I	490	
	PAV = 0.		I	500	
	DO 12 L=LB,LT,NP		I	510	
12	PAV = PAV + POWER(L)		I	520	
	PAV = PAV*PCTPWR*.01/FLOAT(NZ)		I	530	
	CALCULATE RELATIVE CHANNEL FLOW		I	540	
	N = CTYPE(LB)		I	550	
10	FLOW(LB) = ( B3(N)*PAV + B2(N) ) *PAV + B1(N)		I	560	
C	NORMALIZE RELATIVE FLOWS AND CALCULATE ACTUAL FLOW		I	570	
	CALL NORM (FLOW, NP)		I	580	
	AVF = TFLOW/FLOAT(NP)		I	590	
	IF ( QTRKOR ) AVF = AVF*.25		I	600	
	DO 16 L=1,NP		I	610	
16	FLOW(L) = FLOW(L)*AVF		I	620	
C			I	630	
C	-----			I	640
C			I	650	
	CALCULATE QUALITY		I	660	
C			I	670	
	C = AVPWR/HFG*.5		I	680	
	DO 18 LB=1,NP		I	690	
	LT = LB + NDSMNP		I	700	
	DX = C/FLOW(LB)		I	710	
	X = XIN		I	720	

```

      DO 18 L=LB,LT,NP
      X = X + DX* POWER(L)
C
      VOID(L) = 0.
      IF ( X .LE. 0.) GO TO 18
CALCULATE VOID USING MODIFIED ARMAND CORRELATION
      VOID(L) = X*(X*AC2 + AC1)/(X*AC4 + AC3)
18    X = X + DX* POWER(L)
C
      RETURN
      END

```

```

I    730
I    740
I    750
I    760
I    770
I    780
I    790
I    800
I    810
I    820
I    830

```

C=====	J	10
C=====	J	20
C	J	30
C	J	40
SUBROUTINE IJK (L, I, J, K)	J	50
C	J	60
C-----	J	70
C ROUTINE TO CONVERT AN 'L' POSITION INDEX TO (I,J,K) POSITION INDICES	J	80
C-----	J	90
C	J	100
COMMON /GEOM / NODES        , NP                , NDSMNP        , NX	J	110
\$,                NZ                , NXM1                , GX                , GZ	J	120
\$,                NCOL (N04)	J	130
C	J	140
C-----	J	150
C	J	160
K = (L-1)/NP+1	J	170
LL = L-(K-1)*NP	J	180
ISUM = 0	J	190
I = 0	J	200
DO 10 J=1,NX	J	210
I = I+1	J	220
ISUM = ISUM+NCOL(I)	J	230
IF ( ISUM .GE. LL) GO TO 20	J	240
10      CONTINUE	J	250
20      IF ( I .EQ. 1) GO TO 30	J	260
J = LL-ISUM+NCOL(I)	J	270
RETURN	J	280
30      J = LL	J	290
RETURN	J	300
END	J	310

C*=====	K	10
C=====	K	20
C	K	30
C	K	40
SUBROUTINE EDIT (N)	K	50
C	K	60
C-----	K	70
C ROUTINE TO EDIT CHANNEL TYPES, COOLANT FLOW DISTRIBUTION, MINIMUM	K	80
C CRITICAL HEAT FLUX RATIO, EXPOSURE DISTRIBUTION, VOID DISTRIBUTION,	K	90
C RELATIVE POWER DISTRIBUTION, CONTROL ROD POSITIONS, AND FUEL TYPE	K	100
C DISTRIBUTION	K	110
C-----	K	120
C	K	130
COMMON /HEADR / NTITLE(20), AVEXP, NTSTEP	K	140
COMMON /CSINP / BUMASK(N07), VMASK(N08), DFI(N10), DTI(N10)	K	150
\$                SAFI(N10), SATI(N10), NSFFI(N10), NSFTI(N10),	K	160
\$                SDSI(N10), IEXPTS, IVDPTS, ITSETS,	K	170
\$                KSFTI(N10), KSFFI(N10),	K	180
\$                DIMTS, DIMEX, DIMVD, NOTCH(N02),	K	190
\$                TABSET(N01), WORTH(N10)	K	200
INTEGER DIMTS, DIMEX, DIMVD, TABSET	K	210
REAL NSFFI, NSFTI, KSFFI, KSFTI	K	220
COMMON /FUEL / FTYPE(N02), BU(N01), VOID(N01)	K	230
INTEGER FTYPE	K	240
COMMON /NODAL / DF (N01), DT (N01), KSFF (N01)	K	250
\$,                KSFT (N01), TFR (N01), NSF1G (N01)	K	260
\$,                C2 (N01), C4 (N01), C6 (N03)	K	270
\$,                C7 (N01)	K	280
REAL KSFF, KSFT, NSF1G	K	290
COMMON /GEOM / NODES, NP, NDSMNP, NX	K	300
\$,                NZ, NXM1, GX, GZ	K	310
\$,                NCOL (N04)	K	320
COMMON /BC / AF1, AF3, AF5, AF6, AFQ(N04),	K	330
\$                BF1, BF3, BF5, BF6, BFQ(N04),	K	340
\$                AT1, AT3, AT5, AT6, ATQ(N04),	K	350
\$                BT1, BT3, BT5, BT6, BTQ(N04)	K	360

EQUIVALENCE (AFQ(1), AF2), (AFQ(2), AF4),	K	370
\$ (BFQ(1), BF2), (BFQ(2), BF4),	K	380
\$ (ATQ(1), AT2), (ATQ(2), AT4),	K	390
\$ (BTQ(1), BT2), (BTQ(2), BT4)	K	400
COMMON /CONVRG/ MAXI, MAXO, MAXP, EPSO, EPSP, ALPHA, SOR	K	410
LOGICAL SOR	K	420
COMMON /FLUX / POWER(N01), FASTA(N01), THERMA(N01), F(N01)	K	430
COMMON /PROB / INTTYP, QTRKOR, NOHAL, NOCROD, NOVOID, IEDIT(6)	K	440
\$, NOXE, FIRST	K	450
LOGICAL QTRKOR, NOHAL, NOCROD, NOVOID, NOXE, FIRST	K	460
COMMON /WEIGHT/ R, CF, CT, RCF, RCT, B1N, B2N	K	470
COMMON /THERMO/ HFG, XIN, AC1, AC2, AC3, AREA, AVFLUX, AVPWR, PSIA	K	480
\$, B1(4), B2(4), B3(4), CTYPE(N02), FLOW(N02), TFLOW	K	490
\$, PCTPWR	K	500
INTEGER CTYPE	K	510
DIMENSION TEMP(N04)	K	520
LOGICAL OPT	K	530
REAL MCHFR	K	540
C	K	550
C-----	K	560
C	K	570
L2 = 0	K	580
GO TO (2,6,10,14,18,22,32,36,40,60), N	K	590
C	K	600
C=====	K	610
C	K	620
CHANNEL TYPE EDIT	K	630
C	K	640
40 CALL HEADER (1)	K	650
WRITE (6,142) (J,J=1,NX)	K	660
WRITE (6,134)	K	670
DO 42 I=1,NX	K	680
L1 = L2+1	K	690
L2 = L2+NCOL(I)	K	700
42 WRITE (6,132) I, (CTYPE(L),L=L1,L2)	K	710
RETURN	K	720

C		K	730
C=====		K	740
C		K	750
COOLANT FLOW EDIT		K	760
2 CALL HEADER (1)		K	770
WRITE (6,100) TFLOW, (J,J=1,NX)		K	780
WRITE (6,134)		K	790
CALL NORM (FLOW, NP)		K	800
DO 4 I=1,NX		K	810
L1 = L2+1		K	820
L2 = L2+NCOL(I)		K	830
4 WRITE (6,102) I,(FLOW(L),L=L1,L2)		K	840
RETURN		K	850
C		K	860
C=====		K	870
C		K	880
C MCHFR EDIT		K	890
C		K	900
C AREA = COOLANT FLOW AREA PER CHANNEL (FT**2)		K	910
C G = COOLANT FLOW IN CHANNEL (LBM/HR/FT**2)		K	920
C AVPWR = AVERAGE POWER IN NODE (BTU/HR)		K	930
C X = COOLANT QUALITY		K	940
C XIN = INLET QUALITY		K	950
C HFG = HEAT OF VAPORIZATION (BTU/LBM)		K	960
C AVFLUX = AVERAGE HEAT FLUX AT CLAD SURFACE (MULTIPLIED BY A PEAKING		K	970
C FACTOR TO ACCOUNT FOR NON-UNIFORM HEATING OF NODE)		K	980
C (BTU/HR/FT**2)		K	990
C QCRIT = CRITICAL HEAT FLUX DETERMINED FROM JANSSEN-LEVY CORRELATION		K	1000
C (BTU/HR/FT**2)		K	1010
C PRCOR = PRESSURE CORRECTION TERM = 440*(1000-PSIA)		K	1020
C MCHFR = MINIMUM CRITICAL HEAT FLUX RATIO		K	1030
C		K	1040
C-----		K	1050
C		K	1060
C		K	1070
6 IF ( IEDIT(5) .EQ. 0) RETURN		K	1080



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      IF ( PCTPWR .EQ. 0. ) RETURN
      OPT = IEDIT(5) .GT. 0
      CALL HEADER (1)
      IF ( OPT) WRITE (6,104) (J,J=1,NX)
      IF ( OPT) WRITE (6,134)
      C = AVPWR/HFG*.5
      PRCOR = 4.4E5-440.*PSIA
      MCHFR = 1.E10
      CALCULATE CHANNEL FLOW RATES IN LBM/HR
      CALL NORM (FLOW, NP)
      AVF = TFLOW/FLOAT (NP)
      IF ( QTRKOR ) AVF = AVF*.25
      DO 6019 L=1,NP
6019  FLOW(L) = FLOW(L)*AVF
      C
      DO 6002 I=1,NX
      J = 0
      L1 = L2+1
      L2 = L2 + NCOL(I)
      DO 6003 LB=L1,L2
      J = J+1
      LT = LB+NDSMNP
      G = FLOW(LB)/AREA
      DX = C/FLOW(LB)
      X = XIN
      TEMP(J) = 1.E10
      DO 6003 L=LB,LT,NP
      RELPWR = POWER(L)
      X = X + DX* RELPWR
      XCHK = .197-.108E-6*G
      IF ( X-XCHK) 6020,6020,6040
6020  QCRIT = .705E6+.237*G+PRCOR
      GO TO 6100
6040  XCHK = .254-.026E-6*G
      IF ( X-XCHK) 6060,6060,6080
6060  QCRIT = 1.634E6-.27*G-.471E7*X+PRCOR

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      K 1090
      K 1100
      K 1110
      K 1120
      K 1130
      K 1140
      K 1150
      K 1160
      K 1170
      K 1180
      K 1190
      K 1200
      K 1210
      K 1220
      K 1230
      K 1240
      K 1250
      K 1260
      K 1270
      K 1280
      K 1290
      K 1300
      K 1310
      K 1320
      K 1330
      K 1340
      K 1350
      K 1360
      K 1370
      K 1380
      K 1390
      K 1400
      K 1410
      K 1420
      K 1430
      K 1440

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GO TO 6100	K	1450
6080 QCRIT = .605E6-.164*G-.653E6*X+PRCOR	K	1460
6100 CHFR = QCRIT/AVFLUX/RELPWR	K	1470
IF ( CHFR .GE. TEMP(J) ) GO TO 6003	K	1480
TEMP(J) = CHFR	K	1490
IF ( CHFR .GE. MCHFR ) GO TO 6003	K	1500
MCHFR = CHFR	K	1510
LMAX = L	K	1520
6003 X = X + DX* RELPWR	K	1530
IF ( OPT ) WRITE (6,102) I, (TEMP(JJ), JJ=1,J)	K	1540
6002 CONTINUE	K	1550
C	K	1560
CALL IJK (LMAX, I, J, K)	K	1570
WRITE (6,107) MCHFR,I,J,K	K	1580
RETURN	K	1590
C	K	1600
C=====	K	1610
C	K	1620
C EXPOSURE EDIT	K	1630
10 IF ( IEDIT(2) .EQ. 0 ) GO TO 12	K	1640
DO 11 K=1,NZ	K	1650
CALL HEADER (2)	K	1660
WRITE (6,108) K,(J,J=1,NX)	K	1670
WRITE (6,134)	K	1680
DO 11 I=1,NX	K	1690
L1 = L2+1	K	1700
L2 = L2+NCOL(I)	K	1710
11 WRITE (6,110) I, (BU(L),L=L1,L2)	K	1720
IF ( IEDIT(2) .EQ. 1 ) RETURN	K	1730
12 L2 = 0	K	1740
CALL HEADER (2)	K	1750
WRITE (6,109) (J,J=1,NX)	K	1760
WRITE (6,134)	K	1770
DO 13 I=1,NX	K	1780
L1 = L2+1	K	1790
L2 = L2+NCOL(I)	K	1800

J = 0	K	1810
DO 1300 LB=L1,L2	K	1820
J = J+1	K	1830
LT = LB+NDSMNP	K	1840
SUM = 0.	K	1850
DO 1302 L=LB,LT,NP	K	1860
1302 SUM = SUM+BU(L)	K	1870
1300 TEMP(J) = SUM/FLOAT(NZ)	K	1880
13 WRITE (6,110) I,(TEMP(N),N=1,J)	K	1890
RETURN	K	1900
C	K	1910
C=====	K	1920
C	K	1930
C VOID EDIT	K	1940
14 K1 = 1	K	1950
IF ( IEDIT(4) ) 1401,1400,1402	K	1960
1400 RETURN	K	1970
1401 K1 = NZ	K	1980
L2 = NDSMNP	K	1990
1402 DO 16 K=K1,NZ	K	2000
CALL HEADER (1)	K	2010
WRITE (6,112) K,(J,J=1,NX)	K	2020
WRITE (6,134)	K	2030
DO 16 I=1,NX	K	2040
L1 = L2+1	K	2050
L2 = L2+NCOL(I)	K	2060
16 WRITE (6,102) I,(VOID(L),L=L1,L2)	K	2070
RETURN	K	2080
C	K	2090
C=====	K	2100
C	K	2110
C RELATIVE POWER EDIT	K	2120
18 IF ( IEDIT(1) .EQ. 0 ) GO TO 20	K	2130
DO 19 K=1,NZ	K	2140
CALL HEADER (2)	K	2150
WRITE (6,116) K,(J,J=1,NX)	K	2160

	WRITE (6,134)	K	2170
	DO 19 I=1,NX	K	2180
	L1 = L2+1	K	2190
	L2 = L2+NCOL(I)	K	2200
19	WRITE (6,118) I,(POWER(L),L=L1,L2)	K	2210
	IF ( IEDIT(1) .EQ. 1) GO TO 62	K	2220
20	L2 = 0	K	2230
	CALL HEADER (2)	K	2240
	WRITE (6,119) (J,J=1,NX)	K	2250
	WRITE (6,134)	K	2260
	DO 21 I=1,NX	K	2270
	L1 = L2+1	K	2280
	L2 = L2+NCOL(I)	K	2290
	J = 0	K	2300
	DO 2100 LB=L1,L2	K	2310
	J = J+1	K	2320
	LT = LB+NDSMNP	K	2330
	SUM = 0.	K	2340
	DO 2102 L=LB,LT,NP	K	2350
2102	SUM = SUM+POWER(L)	K	2360
2100	TEMP(J) = SUM/FLOAT(NZ)	K	2370
21	WRITE (6,118) I,(TEMP(N),N=1,J)	K	2380
C		K	2390
C-----		K	2400
C		K	2410
C EDIT AVERAGE AXIAL POWER AND VOID		K	2420
62	OPT = .NOT. NOVOID	K	2430
	CALL HEADER (2)	K	2440
	WRITE (6,148)	K	2450
	IF ( OPT) WRITE (6,150)	K	2460
	WRITE (6,152)	K	2470
	IF ( OPT) WRITE (6,154)	K	2480
	L = 0	K	2490
	DO 50 K=1,NZ	K	2500
	V = 0.	K	2510
	P = 0.	K	2520

DO 56 LL=1,NP	K	2530
L = L + 1	K	2540
IF ( NOVOID ) GO TO 56	K	2550
V = V + VOID(L)	K	2560
56 P = P + POWER(L)	K	2570
P = P/FLOAT (NP)	K	2580
IF ( OPT ) V = V/FLOAT (NP)	K	2590
WRITE (6,156) K,P	K	2600
IF ( OPT ) WRITE (6,158) V	K	2610
50 CONTINUE	K	2620
C	K	2630
C-----	K	2640
C	K	2650
C EDIT PEAK NODE AND PEAK ASSEMBLY	K	2660
PEAKN = 0.	K	2670
PEAKA = 0.	K	2680
DO 54 LB=1,NP	K	2690
LT = LB + NDSMNP	K	2700
PAV = 0.	K	2710
DO 52 L=LB,LT,NP	K	2720
IF ( POWER(L) .LE. PEAKN ) GO TO 52	K	2730
PEAKN = POWER(L)	K	2740
LPN = L	K	2750
52 PAV = PAV + POWER(L)	K	2760
IF ( PAV .LE. PEAKA ) GO TO 54	K	2770
PEAKA = PAV	K	2780
LPA = LB	K	2790
54 CONTINUE	K	2800
PEAKA = PEAKA/FLOAT (NZ)	K	2810
CALL IJK (LPN, I, J, K)	K	2820
WRITE (6,160) PEAKN, I, J, K	K	2830
CALL IJK (LPA, I, J, K)	K	2840
WRITE (6,162) PEAKA, I, J	K	2850
RETURN	K	2860
C	K	2870
C=====	K	2880

C		K	2890
C	CONTROL ROD POSITION EDIT	K	2900
C		K	2910
22	CALL HEADER (1)	K	2920
	WRITE (6,120) (J,J=1,NX)	K	2930
	WRITE (6,134)	K	2940
	DO 24 I=1,NX	K	2950
	L1 = L2+1	K	2960
	L2 = L2+NCOL(I)	K	2970
24	WRITE (6,132) I,(NOTCH(L),L=L1,L2)	K	2980
	RETURN	K	2990
C		K	3000
C	=====	K	3010
C		K	3020
C	FUEL TYPE EDIT	K	3030
C		K	3040
32	CALL HEADER (1)	K	3050
	WRITE (6,130) (J,J=1,NX)	K	3060
	WRITE (6,134)	K	3070
	DO 34 I=1,NX	K	3080
	L1 = L2+1	K	3090
	L2 = L2+NCOL(I)	K	3100
34	WRITE (6,132) I,(FTYPE(L),L=L1,L2)	K	3110
	RETURN	K	3120
C		K	3130
C	=====	K	3140
C		K	3150
C	EXPOSURE ARRAY PUNCH	K	3160
36	WRITE (7,136) NTITLE, AVEXP	K	3170
	DO 38 I=1,NX	K	3180
	L1 = L2+1	K	3190
	L2 = L2+NCOL(I)	K	3200
	J = 0	K	3210
	DO 38 LB=L1,L2	K	3220
	LT = LB+NDSMNP	K	3230
	J = J+1	K	3240

	WRITE (7,138) I,J	K	3250
38	WRITE (7,140) (BU(L),L=LB,LT,NP)	K	3260
	RETURN	K	3270
C		K	3280
C=====		K	3290
C		K	3300
C	POWER ARRAY PUNCH	K	3310
60	WRITE (7,144) NTITLE, NTSTEP, AVEXP	K	3320
	WRITE (7,146) POWER	K	3330
	RETURN	K	3340
C		K	3350
C-----		K	3360
C		K	3370
142	FORMAT (1X,'*** CHANNEL TYPE EDIT ***'//5X,'CTYPE (I,J)'//7X, \$J=',I4,15I6)	K	3380
100	FORMAT (1X,'*** RELATIVE FLOW EDIT *** TOTAL FLOW =', \$E10.3,' LBM/HR'//5X,'FLOW (I,J)'//9X,'J=',I3,15I8)	K	3390
102	FORMAT (//15,1X,14F8.2)	K	3400
107	FORMAT (//1X,'MINIMUM CRITICAL HEAT FLUX RATIO = ',F10.3,2X,'AT LO \$CATION (' ,I3,' ,',I3,' ,',I3,' )')	K	3410
104	FORMAT (1X,'*** MINIMUM CRITICAL HEAT FLUX RATIO EDIT ***'//5X,' \$MCHFR (I,J)'//9X,'J=',I3,15I8)	K	3420
108	FORMAT (1X,'*** EXPOSURE EDIT ***'//5X,'EX (I,J,' ,I2,' )'//9X,'J= \$',I3,15I8)	K	3430
110	FORMAT (//15,1X,1P15E8.1)	K	3440
112	FORMAT (1X,'*** VOID FRACTION EDIT ***'//5X,'VOID (I,J,' ,I2,' )'// \$/9X,'J=',I3,15I8)	K	3450
116	FORMAT (1X,'*** RELATIVE POWER EDIT ***'//5X,'P (I,J,' ,I2,' )'//9 \$X,'J=',I3,15I8)	K	3460
118	FORMAT (//15,1X,15F8.3)	K	3470
120	FORMAT (1X,'*** CONTROL ROD POSITIONS ***'//5X,'NOTCH (I,J)'//7 \$X,'J=',I4,14I6)	K	3480
130	FORMAT (1X,'*** FUEL TYPE EDIT ***'//5X,'FTYPE (I,J)'//7X,'J=', \$I4,14I6)	K	3490
132	FORMAT (//15,2X,15I6)	K	3500
134	FORMAT (4X,'I')	K	3510
		K	3520
		K	3530
		K	3540
		K	3550
		K	3560
		K	3570
		K	3580
		K	3590
		K	3600

109	FORMAT (1X,'*** AVERAGE EXPOSURE EDIT ***'//5X,'EX (I,J) '//9X,	K	3610
	\$J=1,I3,I5I8)	K	3620
119	FORMAT (1X,'*** INTEGRATED POWER EDIT ***'//5X,'P (I,J) '//9X,	K	3630
	\$J=1,I3,I5I8)	K	3640
136	FORMAT (20A4/'AVEXP = ',E10.3)	K	3650
138	FORMAT (2I5)	K	3660
140	FORMAT (6E13.3)	K	3670
144	FORMAT (20A4/'TIME STEP #',I4,4X,'AVERAGE CORE EXPOSURE ='	K	3680
	\$,E12.3,' MWD/MT')	K	3690
146	FORMAT (13F6.2)	K	3700
148	FORMAT (11X,'AXIAL AVERAGES'//11X,'===== '//11X,'K POWER')	K	3710
150	FORMAT (1H+,20X,'VOID')	K	3720
152	FORMAT (10X,'-- ----')	K	3730
154	FORMAT (1H+,20X,'----')	K	3740
156	FORMAT (I12,F7.3)	K	3750
158	FORMAT (1H+,F24.2)	K	3760
160	FORMAT (//10X,'PEAK POWER =',F8.3,' AT LOCATION (',I2,',',I2,',',	K	3770
	\$I2,')')	K	3780
162	FORMAT (//10X,'PEAK ASSEMBLY INTEGRATED POWER =',F8.3,' AT LOCATI	K	3790
	\$ON (',I2,',',I2,')')	K	3800
C		K	3810
	END	K	3820



C*	=====	L	10
C	=====	L	20
C		L	30
C		L	40
C	SUBROUTINE BANK (BANKID, NTCH)	L	50
C		L	60
C	-----	L	70
C	ROUTINE TO SET CONTROL ROD POSITIONS BY BANK	L	80
C	-----	L	90
C		L	100
	COMMON /CSINP / BUMASK(N07), VMASK(N08), DFI(N10), DTI(N10)	L	110
\$	SAFI(N10), SATI(N10), NSFFI(N10), NSFTI(N10),	L	120
\$	SDSI(N10), IEXPTS , IVDPTS , ITSETS ,	L	130
\$	KSFTI(N10), KSFFI(N10),	L	140
\$	DIMTS , DIMEX , DIMVD , NOTCH(N02),	L	150
\$	TABSET(N01), WORTH(N10)	L	160
	INTEGER DIMTS, DIMEX, DIMVD, TABSET	L	170
	REAL NSFFI,NSFTI,KSFFI,KSFTI	L	180
	COMMON /CROD / NRODS(6,2), RODIND(N02)	L	190
	INTEGER BANKID, RODIND	L	200
C		L	210
C	-----	L	220
C		L	230
C	RODIND = ARRAY CONTAINING 2-D INDICES INDICATING LOCATIONS OF	L	240
C	NODES WHICH CAN BE RODDED BY THE CONTROL BANKS	L	250
C	NRODS(BANKID,1) = NUMBER OF NODES IN A PLANE WHICH CAN BE RODDED	L	260
C	BY ROD BANK 'BANKID'	L	270
C	NRODS(BANKID,2) = INDEX MARKING THE BEGINNING OF ELEMENTS OF THE	L	280
C	ELEMENTS OF THE RODIND ARRAY ASSIGNED TO	L	290
C	ROD BANK 'BANKID'	L	300
C	NTCH = AXIAL POSITION INDEX OF ROD BANK	L	310
C	NOTCH = ARRAY CONTAINING AXIAL POSITION INDICES FOR ALL	L	320
C	RODDED NODES	L	330
C		L	340
C	-----	L	350
C		L	360

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      I1 = NRODS(BANKID,2)
      I2 = I1+NRODS(BANKID,1)-1
      DO 10 N=I1,I2
      L = RODIND(N)
10    NOTCH(L) = NTCH
      RETURN
      END

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L    370
L    380
L    390
L    400
L    410
L    420
L    430

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C#=====	M	10
C=====	M	20
C	M	30
C	M	40
BLOCK DATA	M	50
C	M	60
C-----	M	70
C	M	80
COMMON /HEADR / NTITLE(20), AVEXP, NTSTEP	M	90
COMMON /CSINP / BUMASK(N07), VMASK(N08), DFI(N10), DTI(N10)	M	100
\$            SAFI(N10), SATI(N10), NSFFI(N10), NSFTI(N10),	M	110
\$            SDSI(N10), IEXPTS, IVDPTS, ITSETS,	M	120
\$            KSFTI(N10), KSFFI(N10),	M	130
\$            DIMTS, DIMEX, DIMVD, NOTCH(N02),	M	140
\$            TABSET(N01), WORTH(N10)	M	150
INTEGER DIMTS, DIMEX, DIMVD, TABSET	M	160
REAL NSFFI, NSFTI, KSFFI, KSFTI	M	170
COMMON /FUEL / FTYPE(N02), BU(N01), VOID(N01)	M	180
INTEGER FTYPE	M	190
COMMON /NODAL / DF (N01), DT (N01), KSFF (N01)	M	200
\$,            KSFT (N01), TFR (N01), NSF1G (N01)	M	210
\$,            C2 (N01), C4 (N01), C6 (N03)	M	220
\$,            C7 (N01)	M	230
REAL KSFF, KSFT, NSF1G	M	240
COMMON /GEOM / NODES, NP, NDSMNP, NX	M	250
\$,            NZ, NXM1, GX, GZ	M	260
\$,            NCOL (N04)	M	270
COMMON /BC / AF1, AF3, AF5, AF6, AFQ(N04),	M	280
\$            BF1, BF3, BF5, BF6, BFQ(N04),	M	290
\$            AT1, AT3, AT5, AT6, ATQ(N04),	M	300
\$            BT1, BT3, BT5, BT6, BTQ(N04)	M	310
EQUIVALENC (AFQ(1), AF2), (AFQ(2), AF4),	M	320
\$            (BFQ(1), BF2), (BFQ(2), BF4),	M	330
\$            (ATQ(1), AT2), (ATQ(2), AT4),	M	340
\$            (BTQ(1), BT2), (BTQ(2), BT4)	M	350
COMMON /CONVRG/ MAXI, MAX0, MAXP, EPS0, EPSP, ALPHA, SOR	M	360

LOGICAL SOR	M	370
COMMON /FLUX / POWER(N01), FASTA(N01), THERMA(N01), F(N01)	M	380
COMMON /PROB / INTTYP, QTRKOR, NOHAL, NOCROD, NOVOID, IEDIT(6)	M	390
\$, NOXE, FIRST	M	400
LOGICAL QTRKOR, NOHAL, NOCROD, NOVOID, NOXE, FIRST	M	410
COMMON /WEIGHT/ R, CF, CT, RCF, RCT, B1N, B2N	M	420
COMMON /THERMO/ HFG, XIN, AC1, AC2, AC3, AREA, AVFLUX, AVPWR, PSIA	M	430
\$, B1(4), B2(4), B3(4), CTYPE(N02), FLOW(N02), TFLOW	M	440
\$, PCTPWR	M	450
INTEGER CTYPE	M	460
COMMON /XENON / PD, PRSA1(N10), PRSA2(N10), SAXE(N10)	M	470
C	M	480
C-----	M	490
C	M	500
DATA F/N01*1./, C2/N01*0./, C4/N01*0./, C6/N03*0./	M	510
\$, C7/N01*0./, BUMASK/N07*0./, VMASK/N08*0./	M	520
\$, DIMVD/N08/, DIMTS/N09/, DIMEX/N07/, VOID/N01*0./, BU/N01*0./	M	530
\$, NODES/N01/, NP/N02/, NDSMNP/N03/, NX/N04/	M	540
\$, NXM1/N05/, NZ/N06/, MAXI/3/, MAXU/6/	M	550
\$, MAXP/30/, EPS0/1.E-6/, EPSP/.5E-2/	M	560
\$, ALPHA/1.65/, FLOW/N02*1./	M	570
\$, AC1/.833/, AC2/.167/, AVEXP/0./	M	580
\$, WORTH/N10*0./, B1/4*1./, B2/4*0./, B3/4*0./	M	590
\$, IEDIT/-1,0,0,1,1,0/, CTYPE/N02*1/	M	600
\$, QTRKOR/.TRUE./, NOCROD/.TRUE./, NOVOID/.TRUE./, PCTPWR/100./	M	610
\$, ITSETS/1/, IEXPTS/1/, IVDPTS/1/, POWER/N01*1./	M	620
\$, NTSTEP/1/, NOHAL/.TRUE./, SOR/.FALSE./	M	630
\$, NOXE/.TRUE./, FIRST/.TRUE./, TABSET/N01*1/, FTYPE/N02*1/	M	640
\$, FASTA/N01*0./, THERMA/N01*0./	M	650
C	M	660
END	M	670